

Department of Energy

§ 962.2

ANNEX B TO APPENDIX G

*Standard Remittance of Advice (RA) for
Payment of Fees*

This Annex should be completed only for SNF burned before midnight between April 6/7, 1983.

I. Identification

A. Purchaser: _____

1. Burnup ¹ (MWDT/MTU)	0–	5,000–	10,000	20,000
	5,000	10,000	20,000	up
2. Initial loading (KgU) (with indicated burnup).				
3. Fee rate (\$/KgU)	80.00	142.00	162.00	184.00
4. Fee (\$).				
5. Total fee (4)				

B. Unit identification (Only one unit may be covered in each report.)

1. Reactor/Facility Name: _____
2. Location: _____
3. Type: _____
4. Capacity: _____
5. Date of Commencement of Operations: _____
6. NRC License No.: _____

II. Fee Calculation

A. Discharged nuclear fuel

B. Nuclear fuel in the reactor core as of midnight of 6/7 April 1983.

Assembly identification	Initial loading (KgU)	Burnup ¹ as of midnight 6/7 April 1983 (MWDT/MTU)	Fee
1..			
2..			
3..			
4..			
5..			
6..			
7..			
8..			
9..			
10..			
11..			
12..			
13..			
14..			
15..			
16..			
17..			
18..			
19..			
20..			
21..			
22..			
23..			
24..			
25..			

AUTHORITY: The Atomic Energy Act of 1954 (42 U.S.C. 2011 *et seq.*); Energy Reorganization Act of 1974 (42 U.S.C. 5801 *et seq.*); Department of Energy Organization Act (42 U.S.C. 7101 *et seq.*); Nuclear Waste Policy Act (Pub. L. 97-425, 96 Stat. 2201).

SOURCE: 52 FR 15940, May 1, 1987, unless otherwise noted.

§ 962.1 Scope.

This part applies only to radioactive waste substances which are owned or produced by the Department of Energy at facilities owned or operated by or for the Department of Energy under the Atomic Energy Act of 1954 (42 U.S.C. 2011 *et seq.*). This part does not apply to substances which are not owned or produced by the Department of Energy.

§ 962.2 Purpose.

The purpose of this part is to clarify the meaning of the term “byproduct material” under section 11e(1) of the Atomic Energy Act of 1954 (42 U.S.C. 2014(e)(1)) for use only in determining the Department of Energy’s obligations under the Resource Conservation and Recovery Act (42 U.S.C. 6901 *et seq.*) with regard to radioactive waste substances owned or produced by the Department of Energy pursuant to the exercise of its responsibilities under the Atomic Energy Act of 1954. This part does not affect materials defined as byproduct material under section 11e(2) of the Atomic Energy Act of 1954 (42 U.S.C. 2014(e)(2)).

¹Please provide (as an attachment) a clear reference to the methodology used to derive the burnup figures (computer codes, etc.) and a clear reference to all data used in the derivation of those figures.

C. Total fee.

(Approved by the Office of Management and Budget under control number 1091-0260)

[48 FR 16599, Apr. 18, 1983; 48 FR 23160, May 24, 1983, as amended at 52 FR 35359, Sept. 18, 1987; 56 FR 67659, Dec. 31, 1991]

PART 962—BYPRODUCT MATERIAL

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§ 962.3 Byproduct material.

(a) For purposes of this part, the term *byproduct material* means any radioactive material (except special nuclear material) yielded in or made radioactive by exposure to the radiation incident to the process of producing or utilizing special nuclear material.

(b) For purposes of determining the applicability of the Resource Conservation and Recovery Act (42 U.S.C. 6901 *et seq.*) to any radioactive waste substance owned or produced by the Department of Energy pursuant to the exercise of its atomic energy research, development, testing and production responsibilities under the Atomic Energy Act of 1954 (42 U.S.C. 2011 *et seq.*), the words “any radioactive material,” as used in paragraph (a) of this section, refer only to the actual radionuclides dispersed or suspended in the waste substance. The nonradioactive hazardous component of the waste substance will be subject to regulation under the Resource Conservation and Recovery Act.

PART 963—YUCCA MOUNTAIN SITE SUITABILITY GUIDELINES

Subpart A—General Provisions

Sec.

963.1 Purpose.

963.2 Definitions.

Subpart B—Site Suitability Determination, Methods, and Criteria

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963.13 Preclosure suitability evaluation method.

963.14 Preclosure suitability criteria.

963.15 Postclosure suitability determination.

963.16 Postclosure suitability evaluation method.

963.17 Postclosure suitability criteria.

AUTHORITY: 42 U.S.C. 2011 *et seq.*; 42 U.S.C. 7101 *et seq.*; 42 U.S.C. 10101, *et seq.*

SOURCE: 66 FR 57336, Nov. 14, 2001, unless otherwise noted.

Subpart A—General Provisions

§ 963.1 Purpose.

(a) The purpose of this part is to establish DOE methods and criteria for determining the suitability of the Yucca Mountain site for the location of a geologic repository. DOE will use these methods and criteria in analyzing the data from the site characterization activities required under section 113 of the Nuclear Waste Policy Act.

(b) This part does not address other information that must be considered and submitted to the President, and made available to the public, by the Secretary under section 114 of the Nuclear Waste Policy Act if the Yucca Mountain site is recommended for development as a geologic repository.

§ 963.2 Definitions.

For purposes of this part:

Applicable radiation protection standard means (1) For the preclosure period, the preclosure numerical radiation dose limits in 10 CFR 63.111(a) and (b) and 63.204; and

(2) For the postclosure period, the postclosure numerical radiation dose limits in 10 CFR 63.311 and 63.321 and radionuclide concentration limits in 10 CFR 63.331.

Barrier means any material, structure or feature that prevents or substantially reduces the rate of movement of water or radionuclides from the Yucca Mountain repository to the accessible environment, or prevents the release or substantially reduces the release rate of radionuclides from the waste. For example, a barrier may be a geologic feature, an engineered structure, a canister, a waste form with physical and chemical characteristics that significantly decrease the mobility of radionuclides, or a material placed over and around the waste, provided that the material substantially delays movement of water or radionuclides.

Cladding is the metallic outer sheath of a fuel rod element; it is generally made of a corrosion resistant zirconium alloy or stainless steel, and is intended to isolate the fuel from the external environment.