

privacy information such as social security numbers, home addresses, or personal phone numbers in their filings unless an NRC regulation or other law requires submission of such information. With respect to copyrighted works, except for limited excerpts that serve the purpose of the adjudicatory filings and would constitute a Fair Use application, participants should not include copyrighted materials in their submission.

For further details with respect to this action, see the application for license amendment dated October 2, 2025 (ADAMS Accession No. ML25275A565).

Attorney for licensee: Jason Zorn, Associate General Counsel, Constellation Energy Generation, LLC 4300 Winfield Road, Warrenville, IL 60555.

NRC Branch Chief: Hipolito Gonzalez.

Dated: November 20, 2025.

For the Nuclear Regulatory Commission.

David Wrona,

Chief, Plant Licensing Branch II-2, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

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NUCLEAR REGULATORY COMMISSION

[NRC-2025-0149]

Draft Interim Staff Guidance: Treatment of Certain Loss-of-Coolant Accident Locations as Beyond-Design-Basis Accidents

AGENCY: Nuclear Regulatory Commission.

ACTION: Draft guidance; request for comment.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is soliciting public comment on its draft Interim Staff Guidance (ISG), DSS-ISG-XX, “Treatment of Certain Loss-of-Coolant Accident Locations as Beyond-Design-Basis Accidents.” The purpose of the ISG is to communicate the key safety principles that would enable the NRC staff to determine that certain break locations that would normally be analyzed as design-basis loss-of-coolant accidents (LOCAs) for light-water reactors can be treated as beyond-design-basis accidents.

DATES: Submit comments by December 24, 2025. Comments received after this date will be considered if it is practical to do so, but the Commission is able to ensure consideration only for comments received before this date.

ADDRESSES: You may submit comments by any of the following methods; however, the NRC encourages electronic comment submission through the Federal rulemaking website:

- *Federal rulemaking website:* Go to <https://www.regulations.gov> and search for Docket ID NRC-2025-0149. Address questions about Docket IDs in *Regulations.gov* to Bridget Curran; telephone: 301-415-1003; email: *Bridget.Curran@nrc.gov*. For technical questions, contact the individuals listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *Mail comments to:* Office of Administration, Mail Stop: TWFN-7-A60M, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, ATTN: Program Management, Announcements and Editing Staff.

For additional direction on obtaining information and submitting comments, see “Obtaining Information and Submitting Comments” in the **SUPPLEMENTARY INFORMATION** section of this document.

FOR FURTHER INFORMATION CONTACT: Carolyn Lauron, telephone: 301-415-2736; email: *Carolyn.Lauron@nrc.gov* or Vic Cusumano, telephone: 301-415-4011; email: *Victor.Cusumano@nrc.gov*, both in the Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

SUPPLEMENTARY INFORMATION:

I. Obtaining Information and Submitting Comments

A. Obtaining Information

Please refer to Docket ID NRC-2025-0149 when contacting the NRC about the availability of information for this action. You may obtain publicly available information related to this action by any of the following methods:

- *Federal Rulemaking Website:* Go to <https://www.regulations.gov> and search for Docket ID NRC-2025-0149.

- *NRC’s Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly available documents online in the ADAMS Public Documents collection at <https://www.nrc.gov/reading-rm/adams.html>. To begin the search, select “Begin Web-based ADAMS Search.” For problems with ADAMS, please contact the NRC’s Public Document Room (PDR) reference staff at 1-800-397-4209, at 301-415-4737, or by email to *PDR.Resource@nrc.gov*. The draft ISG for the “Treatment of Certain Loss-of-Coolant Accident Locations as Beyond-Design-Basis Accidents” is available in ADAMS under Accession No. ML25043A335.

- *NRC’s PDR:* The PDR, where you may examine and order copies of publicly available documents, is open by appointment. To make an appointment to visit the PDR, please send an email to *PDR.Resource@nrc.gov* or call 1-800-397-4209 or 301-415-4737, between 8 a.m. and 4 p.m. eastern time (ET), Monday through Friday, except Federal holidays.

B. Submitting Comments

The NRC encourages electronic comment submission through the Federal rulemaking website (<https://www.regulations.gov>). Please include Docket ID NRC-2025-0149 in your comment submission.

The NRC cautions you not to include identifying or contact information that you do not want to be publicly disclosed in your comment submission. The NRC will post all comment submissions at <https://www.regulations.gov> as well as enter the comment submissions into ADAMS. The NRC does not routinely edit comment submissions to remove identifying or contact information.

If you are requesting or aggregating comments from other persons for submission to the NRC, then you should inform those persons not to include identifying or contact information that they do not want to be publicly disclosed in their comment submission. Your request should state that the NRC does not routinely edit comment submissions to remove such information before making the comment submissions available to the public or entering the comment into ADAMS.

II. Background

The emergency core cooling system (ECCS) performance requirements in section 50.46 of title 10 of the *Code of Federal Regulations* (10 CFR), “Acceptance criteria for emergency core cooling systems for light-water nuclear power reactors,” assume as their starting point that a LOCA has occurred. Such an approach is called “non-mechanistic” and presumes reactor coolant pressure boundary rupture without regard to cause. Mechanistic (*i.e.*, based on physical processes or phenomena) rationales for determining that certain LOCAs are unlikely to occur have generally not been accepted.

The NRC, however, has accepted mechanistic rationales for dispositioning certain phenomena for limited purposes. For example, the dynamic effects of pipe ruptures can be excluded from consideration in the design bases under 10 CFR part 50, Appendix A, “General Design Criteria [(GDC)] for Nuclear Power Plants,” GDC

4 if certain conditions are met. Specifically, the NRC needs to review and approve analyses that demonstrate that the probability of fluid system piping rupture is “extremely low” under conditions consistent with the design basis for the piping. The determination that the probability of pipe ruptures is extremely low under GDC 4 is only for the analysis of dynamic effects and does not apply to the design-basis LOCA spectrum usually used to calculate ECCS or containment performance, among other aspects of system, structure, or component design. The NRC has nonetheless begun considering other aspects of reactor design for which engineering analysis methods have developed to a point that mechanistic considerations may be employed to exclude some LOCAs from the design basis while continuing to maintain high level of probability that the emergency core cooling function will be accomplished. Other design-basis analyses that depend on the results of ECCS analyses may also be affected by this approach. Further, the NRC has begun rulemaking efforts to apply relaxed analytical methods to certain classes of LOCAs.

The NRC is currently considering circumstances under which an alternative interpretation of the design-basis LOCA spectrum may be found to be acceptable. For some applications now under review and anticipated to be submitted in the near to medium term, designers have sought to holistically reduce LOCA risks (e.g., reduced numbers of penetrations, larger volumes of water above the core, extended coping times, passive cooling systems). In consideration of design-specific information, the NRC can review justifications that design-basis LOCAs need not be postulated at all conceivable locations.

This draft guidance describes the mechanistic considerations that the NRC staff may consider in determining whether an applicant has proposed an adequately protective design-basis LOCA spectrum.

Dated: November 19, 2025.

For the Nuclear Regulatory Commission.

Victor Cusumano,

*Deputy Director, Division of Safety Systems,
Office of Nuclear Reactor Regulation.*

[FR Doc. 2025-20707 Filed 11-21-25; 8:45 am]

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SECURITIES AND EXCHANGE COMMISSION

[Release No. 34-104220]

Order Cancelling Registration of Municipal Advisor, Melio & Company, LLC, Pursuant to Section 15B(c)(3) of the Securities Exchange Act of 1934

November 19, 2025.

Melio & Company, LLC (CIK No. 1620465, SEC File No. 866-00100-00), hereinafter referred to as the “registrant,” is registered with the Securities and Exchange Commission (the “Commission”) as a municipal advisor pursuant to Sections 15B(a)(1)(B) and 15B(a)(2) of the Securities Exchange Act of 1934 (the “Act”).

On September 23, 2025, a Notice of Intention to Cancel Registration of a Certain Municipal Advisor, including the registrant, was published in the **Federal Register** (Securities and Exchange Commission Release No. 34-103999). The notice gave interested persons an opportunity to request a hearing and stated that an order or orders cancelling the registration would be issued unless a hearing was ordered. No request for a hearing has been filed by any persons (including registrant), and the Commission has not ordered a hearing.

Pursuant to Section 15B(c)(3) of the Act, the Commission has found that registrant is no longer in existence or has ceased to do business as a municipal advisor.

Accordingly,

It is ordered, pursuant to Section 15B(c)(3) of the Act, that the registration of Melio & Company, LLC (CIK No. 1620465, SEC File No. 866-00100-00) be, and hereby is, cancelled.

For the Commission, by the Office of Municipal Securities, pursuant to delegated authority.¹

Sherry R. Haywood,

Assistant Secretary.

[FR Doc. 2025-20689 Filed 11-21-25; 8:45 am]

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SECURITIES AND EXCHANGE COMMISSION

[Release No. 34-104231; File No. SR-PHLX-2025-54]

Self-Regulatory Organizations; Nasdaq PHLX LLC.; Notice of Filing and Immediate Effectiveness of Proposed Rule Change To Amend Rule Equity 7, Section 3 (Nasdaq PSX Fees) To Establish Port and Disaster Recovery Fees for Newly Added CORE FIX Entry Ports and Remove the Temporary Fee Waiver Language Pertaining to OUCH 5.0

November 19, 2025.

Pursuant to Section 19(b)(1)² of the Securities Exchange Act of 1934 (the “Act”)³ and Rule 19b-4 thereunder,³ notice is hereby given that, on September 25, 2025, Nasdaq PHLX LLC (“Phlx” or “Exchange”) filed with the Securities and Exchange Commission (the “Commission”) the proposed rule change as described in Items I and II below, which Items have been prepared by the Exchange. The Commission is publishing this notice to solicit comments on the proposed rule change from interested persons.

I. Self-Regulatory Organization’s Statement of the Terms of Substance of the Proposed Rule Change

The Exchange proposes to amend Rule Equity 7, Section 3 (Nasdaq PSX Fees) to establish port and disaster recovery fees for newly added CORE FIX entry ports and remove the temporary fee waiver language pertaining to OUCH 5.0, as described further below.

The text of the proposed rule change is available on the Exchange’s website at <https://listingcenter.nasdaq.com/rulebook/bx/rulefilings>, and at the principal office of the Exchange.

II. Self-Regulatory Organization’s Statement of the Purpose of, and the Statutory Basis for, the Proposed Rule Change

In its filing with the Commission, the Exchange included statements concerning the purpose of and basis for the proposed rule change and discussed any comments it received on the proposed rule change. The text of these statements may be examined at the places specified in Item IV below. The Exchange has prepared summaries, set forth in sections A, B, and C below, of the most significant aspects of such statements.

¹ 15 U.S.C. 78s(b)(1).

² 15 U.S.C. 78a.

³ 17 CFR 240.19b-4.