

LICENSE AMENDMENT ISSUANCE(S)—Continued

Vistra Operations Company LLC; Comanche Peak Nuclear Power Plant, Unit Nos. 1 and 2; Somervell County, TX

Docket No(s)	50-445, 50-446.
Amendment Date	November 16, 2020.
ADAMS Accession No	ML20168A924.
Amendment No(s)	176 (Unit 1) and 176 (Unit 2).
Brief Description of Amendment(s)	The amendments revised Technical Specification 3.4.15, "RCS [Reactor Coolant System] Leakage Detection Instrumentation," to align with the Standard Technical Specifications for Westinghouse Plants and incorporated the changes made by Technical Specifications Task Force (TSTF) Traveler TSTF-513, Revision 3, "Revise PWR [Pressurized-Water Reactor] Operability Requirements and Actions for RCS Leakage Instrumentation."
Public Comments Received as to Proposed NSHC (Yes/No).	No.

Wolf Creek Nuclear Operating Corporation; Wolf Creek Generating Station, Unit 1; Coffey County, KS

Docket No(s)	50-482.
Amendment Date	December 7, 2020.
ADAMS Accession No.	ML20276A149.
Amendment No(s)	226.
Brief Description of Amendment(s)	The amendment revised Technical Specification 5.5.16, "Containment Leakage Rate Testing Program," to extend the Type A and Type C leak rate test frequencies. Specifically, the change allows the extension of the Type A integrated leakage rate test containment test interval to 15 years and the extension of the Type C local leakage rate test interval to 75 months.
Public Comments Received as to Proposed NSHC (Yes/No).	No.

Dated: December 18, 2020.

For the Nuclear Regulatory Commission.

Caroline L. Carusone,

Deputy Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. 2020-28442 Filed 12-28-20; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[NRC-2012-0110]

Acceptability of Probabilistic Risk Assessment Results for Risk-Informed Activities

AGENCY: Nuclear Regulatory Commission.

ACTION: Regulatory guide; issuance.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is issuing Revision 3 to Regulatory Guide (RG) 1.200, "Acceptability of Probabilistic Risk Assessment Results for Risk-Informed Activities." Revision 3 describes one acceptable approach for determining whether a base probabilistic risk assessment (PRA), in total or the portions that are used to support an application, is acceptable to provide confidence in the results, such that the PRA can be used in regulatory decisionmaking for light-water reactors (LWRs). When used in support of an application, the use of this RG will obviate the need for an in-depth review of the base PRA by NRC reviewers, allowing them to focus their review on key assumptions and areas identified by peer reviewers.

DATES: Revision 3 to RG 1.200 is available on December 29, 2020.

ADDRESSES: Please refer to Docket ID NRC-2012-0110 when contacting the NRC about the availability of information regarding this document. You may obtain publicly available information related to this document using any of the following methods:

- *Federal Rulemaking Website:* Go to <https://www.regulations.gov> and search for Docket ID NRC-2012-0110. Address questions about Docket IDs in *Regulations.gov* to Jennifer Borges; telephone: 301-287-9127; email: Jennifer.Borges@nrc.gov. For technical questions, contact the individuals listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly available documents online in the ADAMS Public Documents collection at <https://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced (if it is available in ADAMS) is provided the first time that it is mentioned in this document.

- *Attention:* The PDR, where you may examine and order copies of public documents is currently closed. You may submit your request to the PDR via email at pdr.resource@nrc.gov or call 1-800-397-4209 between 8:00 a.m. and 4:00 p.m. (EST), Monday through Friday, except Federal holidays.

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FOR FURTHER INFORMATION CONTACT:

Anders Gilbertson, telephone: 301-415-1541, email: Anders.Gilbertson@nrc.gov, and Harriet Karagiannis, telephone: 301-415-2493, email:

Harriet.Karagiannis@nrc.gov. Both are staff of the Office of Nuclear Regulatory Research, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

SUPPLEMENTARY INFORMATION:

I. Discussion

The NRC is issuing a revision to an existing guide in the NRC's "Regulatory Guide" series. This series was developed to describe and make available to the public information regarding methods that are acceptable to the NRC staff for implementing specific parts of the agency's regulations, techniques that the NRC staff uses in evaluating specific issues or postulated events, and data that the NRC staff needs in its review of applications for permits and licenses. Regulatory guides are not NRC regulations and compliance with them is not required. Methods and solutions that differ from those set forth in RGs are acceptable if supported by a basis for the issuance or continuance of a permit or license by the Commission.

Revision 3 to RG 1.200 was issued with a temporary identification of Draft Regulatory Guide, DG-1362 (ADAMS Accession No. ML19308B636). RG 1.200 (Revision 3) describes one acceptable approach for determining whether the base PRA, in total or the portions that are used to support an application, is

sufficient to provide confidence in the results, such that the PRA can be used in regulatory decisionmaking for LWRs. Also, it addresses new industry guidance and enhancements identified since the last revision was issued in March 2009. Specifically, this revision endorses, with staff clarifications and exceptions, the American Society of Mechanical Engineers (ASME) and American Nuclear Society (ANS) Standard ASME/ANS RA-Sa-2009, "Standard for Level 1/ Large Early Release Frequency Probabilistic Risk Assessment for Nuclear Power Plant Applications"; the ASME/ANS standard ASME/ANS RA-S Case 1 for seismic PRA, "Case for ASME/ANS RA-Sb-2013 Standard for Level 1/ Large Early Release Frequency Probabilistic Risk Assessment of Nuclear Power Plant Applications"; Nuclear Energy Institute (NEI) 17-07, Revision 2, "Performance of PRA Peer Reviews Using the ASME/ANS PRA Standard" (ADAMS Accession No. ML19241A615); and Pressurized Water Reactor Owners Group (PWROG) report PWROG-19027-NP, Revision 2, "Newly Developed Method Requirements and Peer Review" (ADAMS Accession No. ML20213C660). This revision of the RG further provides for a peer review of newly developed methods, clarifies the process for determining how to classify changes to a PRA, provides definitions related to newly developed methods and other PRA terms, and enhances guidance related to key assumptions and sources of uncertainty.

II. Additional Information

The NRC published a notice of the availability of DG-1362 in the **Federal Register** on July 1, 2020 (85 FR 39599) for a 30-day public comment period. The public comment period closed on July 31, 2020. Public comments on DG-1362 and the staff responses to the public comments are available in ADAMS under Accession No. ML20238B873. Revision 3 to RG 1.200 may be found in ADAMS under Accession No. ML20238B871.

III. Congressional Review Act

This RG is a rule as defined in the Congressional Review Act (5 U.S.C. 801-808). However, the Office of Management and Budget has not found it to be a major rule as defined in the Congressional Review Act.

IV. Backfitting, Forward Fitting, and Issue Finality

This RG provides one acceptable approach for determining whether the base PRA, in total or the portions that are used to support an application, is

sufficient to provide confidence in the results, such that the PRA can be used in regulatory decisionmaking for LWRs. Issuance of this RG does not constitute backfitting as defined in section 50.109 of title 10 of the *Code of Federal Regulations* (10 CFR), "Backfitting," and as described in NRC Management Directive 8.4, "Management of Backfitting, Forward Fitting, Issue Finality, and Information Requests" (ADAMS Accession No. ML18093B087); does not constitute forward fitting as that term is defined and described in Management Directive 8.4; and does not affect the issue finality of any approval issued under 10 CFR part 52, "Licenses, Certificates, and Approvals for Nuclear Power Plants." As explained in this RG, applicants and licensees are not required to comply with the positions set forth in this RG.

Dated: December 21, 2020.

For the Nuclear Regulatory Commission.

Robert G. Roche-Rivera,

Acting Chief, Regulatory Guidance and Generic Issues Branch, Division of Engineering, Office of Nuclear Regulatory Research.

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NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-338-SLR and 50-339-SLR; ASLBP No. 21-970-01-SLR-01]

Virginia Electric and Power Company; Establishment of Atomic Safety and Licensing Board

Pursuant to delegation by the Commission, *see* 37 FR 28,710 (Dec. 29, 1972), and the Commission's regulations, *see, e.g.*, 10 CFR 2.104, 2.105, 2.300, 2.309, 2.313, 2.318, 2.321, notice is hereby given that an Atomic Safety and Licensing Board (Board) is being established to preside over the following proceeding:

Virginia Electric and Power Company (North Anna Power Station, Units 1 and 2)

This proceeding involves an application seeking a twenty-year subsequent license renewal of Renewed Facility Operating License Nos. NPF-4 and NPF-7, which currently authorize Virginia Electric and Power Company to operate the North Anna Power Company, Units 1 and 2, located in Louisa, Virginia, until, respectively, April 1, 2038 and August 21, 2040. In response to a notice published in the **Federal Register** announcing the opportunity to request a hearing, *see* 85 FR 65,438 (Oct. 15, 2020), a hearing

request was filed on December 14, 2020 on behalf of Beyond Nuclear, Sierra Club, and Alliance for Progressive Virginia.

The Board is comprised of the following Administrative Judges: G. Paul Bollwerk, III, Chairman, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; Nicholas G. Trikouros, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; Dr. Gary S. Arnold, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001

All correspondence, documents, and other materials shall be filed in accordance with the NRC E-Filing rule. *See* 10 CFR 2.302.

Rockville, Maryland. December 21, 2020.

Edward R. Hawkens,

Chief Administrative Judge, Atomic Safety and Licensing Board Panel.

[FR Doc. 2020-28634 Filed 12-28-20; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[NRC-2020-0237]

Considerations for Estimating Site-Specific Probable Maximum Precipitation at Nuclear Power Plants in the United States of America

AGENCY: Nuclear Regulatory Commission.

ACTION: Draft NUREG; request for comment.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is issuing for public comment a draft NUREG, knowledge management NUREG, NUREG/KM-0015, "Considerations for Estimating Site-Specific Probable Maximum Precipitation at Nuclear Power Plants in the United States of America." The NRC Staff and Oak Ridge National Laboratory have prepared a reference document summarizing recent lessons-learned in connection with a review of the site-specific probable maximum precipitation (SSPMP) estimates used by some nuclear power plant owners and operators in connection with a recent re-evaluation of external flooding at their respective project sites.

DATES: Submit comments by March 1, 2021. Comments received after this date will be considered if it is practical to do so, but the Commission is able to ensure consideration only for comments received before this date.