

Brief Description of Amendment(s) .....	The amendments transitioned the Hatch Fire Protection Program from 10 CFR Sections 50.48(a) and (b) to 10 CFR 50.48(c), National Fire Protection Association (NFPA) 805, "Performance-Based Standard for Fire Protection for Light Water Reactor Electric Generating Plants," 2001 Edition.
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**Southern Nuclear Operating Company, Inc.; Vogtle Electric Generating Plant, Units 3 and 4; Burke County, GA**

Docket No(s) .....	52-025, 52-026.
Amendment Date .....	August 11, 2020.
ADAMS Accession No. of Issuance of Amendment Letter .....	ML20196L674.
Amendment No(s) .....	183, 181.
Brief Description of Amendment(s) .....	The amendments consisted of changes to the following Technical Specifications (TS): (A) Surveillance Requirement (SR) 3.7.6.3 frequency revised for Main Control Room Emergency Habitability System (VES) operation and deleted SR 3.7.6.9 which verifies the self-contained pressure regulating valve in each VES air delivery flow path is operable in accordance with the Inservice Testing Program; (B) SR 3.3.8.2 for channel calibration and SR 3.3.8.3 for engineered safety feature response time revised to include a Note excluding neutron detectors; (C) TS 5.5.3, "Inservice Testing Program," revised to replace existing detail with a reference to fulfilling the requirements of 10 CFR 50.55a(f); (D) TS 5.5.9, "System Level OPERABILITY Testing Program," revised for appropriate wording consistency and appropriate reference to the Updated Final Safety Analysis Report; and (E) TS 3.4.9, "RCS [Reactor Coolant System] Leakage Detection Instrumentation," Applicability Note 2 revised to consistently identify the applicable power level.

**Tennessee Valley Authority; Watts Bar Nuclear Plant, Unit 2; Rhea County, TN**

Docket No(s) .....	50-391.
Amendment Date .....	August 10, 2020.
ADAMS Accession No. of Issuance of Amendment Letter .....	ML20156A018.
Amendment No(s) .....	40.
Brief Description of Amendment(s) .....	The amendment revised Watts Bar Nuclear Plant, Unit 2, Technical Specification (TS) 3.4.17, "Steam Generator (SG) Tube Integrity"; TS 5.7.2.12, "Steam Generator (SG) Program"; and TS 5.9.9, "Steam Generator Tube Inspection Report," to allow the use of Westinghouse leak limiting non-nickel banded Alloy 800 sleeves to repair degraded SG tubes as an alternative to plugging the tubes.

**Tennessee Valley Authority; Watts Bar Nuclear Plant, Units 1 and 2; Rhea County, TN**

Docket No(s) .....	50-390, 50-391.
Amendment Date .....	August 19, 2020.
ADAMS Accession No. of Issuance of Amendment Letter .....	ML20167A148.
Amendment No(s) .....	136 (Unit 1) and 41 (Unit 2).
Brief Description of Amendment(s) .....	The amendments deleted Technical Specification 3.8.1, "AC [Alternating Current] Sources—Operating," Surveillance Requirement 3.8.1.22 to verify the operability of the automatic transfer from a Unit Service Station Transformer to a Common Station Service Transformer A or B at the associated unit board.

Dated: August 28, 2020.  
For the Nuclear Regulatory Commission.

**Gregory F. Suber,**  
*Deputy Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.*

[FR Doc. 2020-19416 Filed 9-4-20; 8:45 am]

**BILLING CODE 7590-01-P**

**NUCLEAR REGULATORY COMMISSION**

[Docket No. 11006377; NRC-2020-0186]

**EnergySolutions Services, Inc.**

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Export license application; opportunity to provide comments, request a hearing, and petition for leave to intervene.

**SUMMARY:** The Nuclear Regulatory Commission (NRC) received and is considering issuing an export license (XW026) requested by EnergySolutions Services Inc. (ESSI). On July 27, 2020, ESSI filed an application with the NRC for a license to export low-level radioactive waste to Mexico. The NRC is providing notice of the opportunity to comment, request a hearing, and petition to intervene on ESSI's application.

**DATES:** Submit comments by October 8, 2020. Comments received after this date will be considered if it is practical to do so, but the Commission is able to ensure consideration only for comments received before this date. A request for a hearing or petition for leave to intervene must be filed by October 8, 2020.

**ADDRESSES:** You may submit comments by any of the following methods:

- *Federal Rulemaking website:* Go to <https://www.regulations.gov> and search for Docket ID NRC-2020-0186. Address questions about Docket IDs in *Regulations.gov* to Jennifer Borges; telephone: 301-287-9127; email: [Jennifer.Borges@nrc.gov](mailto:Jennifer.Borges@nrc.gov). For technical

questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *Email comments to:* [hearing.docket@nrc.gov](mailto:hearing.docket@nrc.gov). If you do not receive an automatic email reply confirming receipt, then contact us at 301-415-1677.
- *Fax comments to:* Secretary, U.S. Nuclear Regulatory Commission at 301-415-1101.

- *Mail comments to:* Secretary, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, ATTN: Rulemakings and Adjudications Staff.

For additional direction on obtaining information and submitting comments, see “Obtaining Information and Submitting Comments” in the **SUPPLEMENTARY INFORMATION** section of this document.

**FOR FURTHER INFORMATION CONTACT:** Gary Langlie, Office of International Programs, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; telephone: 301-287-9076, email: [Gary.Langlie@nrc.gov](mailto:Gary.Langlie@nrc.gov).

**SUPPLEMENTARY INFORMATION:**

**I. Obtaining Information and Submitting Comments**

*A. Obtaining Information*

Please refer to NRC-2020-0186 or Docket No. 11006377 when contacting the NRC about the availability of information for this action. You may obtain publicly-available information related to this action by any of the following methods:

- *Federal rulemaking website:* Go to <https://www.regulations.gov> and search for Docket ID NRC-2020-0186.
- *NRC’s public website:* Go to <https://www.nrc.gov> and search for XW026, Docket No. 11006377, or Docket ID NRC-2020-0186.
- *NRC’s Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly-available documents online in the ADAMS Public Documents collection at <https://www.nrc.gov/reading-rm/>

*adams.html*. To begin the search, select “Begin Web-based ADAMS Search.” For problems with ADAMS, please contact the NRC’s Public Document Room reference staff at 1-800-397-4209, 301-415-4737, or by email to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov). The export license application from ESSI is available in ADAMS under Accession No. ML20210M325.

*B. Submitting Comments*

Please include Docket ID NRC-2020-0186 or Docket No. 11006377 in your comment submission. The NRC cautions you not to include identifying or contact information that you do not want to be publicly disclosed in your comment submission. The NRC will post all comment submissions at <https://www.regulations.gov> as well as enter the comment submissions into ADAMS. The NRC does not routinely edit comment submissions to remove identifying or contact information.

If you are requesting or aggregating comments from other persons for submission to the NRC, then you should inform those persons not to include identifying or contact information that they do not want to be publicly disclosed in their comment submission. Your request should state that the NRC does not routinely edit comment submissions to remove such information before making the comment submissions available to the public or entering the comment into ADAMS.

**II. Discussion**

On July 27, 2020, NRC received an application from EnergySolutions Services Inc. (ESSI) requesting a specific license (XW026) to export radioactive waste in the form of metals, dry active material, and plastic spent ion exchange resins and liquids to Mexico (ADAMS Accession No. ML20210M325).

In accordance with paragraph 110.70(b) of title 10 of the *Code of Federal Regulations* (10 CFR) the NRC is providing notice of the receipt of the application; providing the opportunity

to submit written comments concerning the application; and providing the opportunity to request a hearing or petition for leave to intervene, for a period of 30 days after publication of this notice in the **Federal Register**. A hearing request or petition for leave to intervene must include the information specified in 10 CFR 110.82(b). Any request for hearing or petition for leave to intervene shall be served by the requestor or petitioner in accordance with 10 CFR 110.89(a), either by delivery, by mail, or filed with the NRC electronically in accordance with the NRC’s E-Filing rule (72 FR 49139; August 28, 2007, as amended at 77 FR 46562; August 3, 2012). Detailed guidance on making electronic submissions may be found in the Guidance for Electronic Submissions to the NRC and on the NRC website at <https://www.nrc.gov/site-help/e-submittals.html>.

To comply with the procedural requirements of E-Filing, at least 10 days prior to the filing deadline, the participant should contact the Office of the Secretary by email at [hearing.docket@nrc.gov](mailto:hearing.docket@nrc.gov), or by telephone at 301-415-1677, to (1) request a digital identification (ID) certificate, which allows the participant (or its counsel or representative) to digitally sign submissions and access the E-Filing system for any proceeding in which it is participating; and (2) advise the Secretary that the participant will be submitting a petition or other adjudicatory document (even in instances in which the participant, or its counsel or representative, already holds an NRC-issued digital ID certificate). Based upon this information, the Secretary will establish an electronic docket for the hearing in this proceeding if the Secretary has not already established an electronic docket.

The information concerning this application for an export license follows.

**NRC EXPORT LICENSE APPLICATION**

**Application Information**

Name of applicant .....	EnergySolutions Services, Inc.
Date of Application .....	July 21, 2020.
Date Received .....	July 27, 2020.
Application No. ....	XW026.
Docket No. ....	11006377.
ADAMS Accession No. ....	ML20210M325.

**Description of Material**

Material Type .....	Radioactively contaminated material and/or waste in the form of metals, dry active waste or material, such as wood, paper, and plastic, and spent ion exchange resins and liquids, in the form of aqueous and organic based fluids.
Total Quantity .....	Not to exceed 653.96 terabecquerels (TBq).

## NRC EXPORT LICENSE APPLICATION—Continued

End Use .....	Storage or disposal by the original generator.
Country of Destination .....	Mexico.

Dated: September 1, 2020.

For the Nuclear Regulatory Commission.

**David L. Skeen,**

*Deputy Director, Office of International Programs.*

[FR Doc. 2020-19734 Filed 9-4-20; 8:45 am]

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## NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-454; 50-455; 50-456; 50-457; NRC-2020-0206]

**Exelon Generation Company, LLC;  
Byron Station, Unit Nos. 1 and 2;  
Braidwood Station, Units 1 and 2**

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Exemption; issuance.

**SUMMARY:** The U.S. Nuclear Regulatory Commission (NRC) is issuing an exemption in response to a September 30, 2019, request from Exelon Generation Company, LLC (Exelon) from regulatory requirements to allow Byron Station, Unit Nos. 1 and 2, and Braidwood Station, Units 1 and 2, to use an alternative methodology for determining reactor coolant system pressure-temperature limits. The methodology is described in AREVA NP Topical Report BAW-2308, Revisions 1-A and 2-A, "Initial RT<sub>NDT</sub> of Linde 80 Weld Materials."

**DATES:** The exemption was issued on August 31, 2020.

**ADDRESSES:** Please refer to Docket ID NRC-2020-0206 when contacting the NRC about the availability of information regarding this document. You may obtain publicly-available information related to this document using any of the following methods:

- *Federal Rulemaking Website:* Go to <https://www.regulations.gov> and search for Docket ID NRC-2020-0206. Address questions about Docket IDs in *Regulations.gov* to Jennifer Borges; telephone: 301-287-9127; email: [Jennifer.Borges@nrc.gov](mailto:Jennifer.Borges@nrc.gov). For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly-available documents online in the ADAMS Public Documents collection at <https://www.nrc.gov/reading-rm/>

*adams.html*. To begin the search, select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room reference staff at 1-800-397-4209, 301-415-4737, or by email to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov). The ADAMS accession number for each document referenced (if it is available in ADAMS) is provided the first time that it is mentioned in this document.

**FOR FURTHER INFORMATION CONTACT:** Joel S. Wiebe, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington DC 20555-0001; telephone: 301-415-6606, email: [Joel.Wiebe@nrc.gov](mailto:Joel.Wiebe@nrc.gov).

**SUPPLEMENTARY INFORMATION:** The text of the exemption is attached.

Dated: September 1, 2020.

For the Nuclear Regulatory Commission.

**Joel S. Wiebe,**

*Senior Project Manager, Licensing Projects Branch III, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.*

### Attachment—Exemption

## NUCLEAR REGULATORY COMMISSION

**Docket Nos. 50-454; 50-455; 50-456; 50-457**

**Exelon Generation Company, LLC;  
Byron Station, Unit Nos. 1 and 2, and  
Braidwood Station, Units 1 and 2;  
Exemption**

### I. Background

Exelon Generation Company, LLC (Exelon, the licensee), holds Renewed Facility Operating License Nos. NPF-37 and NPF-66, which authorize operation of the Byron Station, Unit Nos. 1 and 2 (Byron), a pressurized-water reactor facility, located in Ogle County, Illinois and Renewed Facility Operating License Nos. NPF-72 and NPF-77, which authorize operation of the Braidwood Station, Units 1 and 2 (Braidwood), a pressurized-water reactor facility, located in Will County, Illinois. The licenses, among other things, subject the facilities to all rules, regulations, and orders of the U.S. Nuclear Regulatory Commission (NRC, the Commission) now or hereafter in effect.

By letter dated September 30, 2019 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML19275E307), Exelon requested exemptions from specific

requirements of Title 10 of the *Code of Federal Regulations* (10 CFR), Part 50, Section 50.61, "Fracture Toughness Requirements for Protection Against Pressurized Thermal Shock Events," and 10 CFR part 50, Appendix G, "Fracture Toughness Requirements," for Braidwood and Byron. The requested exemptions from these requirements would allow use of an alternative methodology to determine reactor coolant system pressure-temperature limits. The new methodology that Exelon intends to use is described in AREVA NP Topical Report BAW-2308, Revisions 1-A and 2-A, "Initial RT<sub>NDT</sub> of Linde 80 Weld Materials" (BAW-2308) (ADAMS Accession No. ML032380449 and ML081270388). BAW-2308 was approved for referencing in plant specific license amendments by NRC letters dated August 4, 2005 (ADAMS Accession No. ML052070408), and March 24, 2008 (ADAMS Accession No. ML080770349).

### II. Request/Action

Pursuant to 10 CFR, Part 50, Section 50.61, "Fracture Toughness Requirements for Protection Against Pressurized Thermal Shock Events," and 10 CFR part 50, Appendix G, "Fracture Toughness Requirements," the Commission's regulations establish specific fracture toughness requirements for nuclear power plant reactor pressure vessels (RPVs). In its letter dated September 30, 2019, Exelon requested exemptions from these requirements to allow use of an alternative methodology described in BAW-2308. BAW-2308 provides an alternate methodology for evaluating the integrity of certain RPV beltline welds, at Braidwood and Byron. The methodology described in BAW-2308, utilized fracture toughness test data based on the use of the 1997 and 2002 editions of American Society for Testing and Materials (ASTM) Standard Test Method E 1921, "Standard Test Method for Determination of Reference Temperature T<sub>0</sub>, for Ferritic Steels in the Transition Range," and American Society for Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Code Case N-629, "Use of Fracture Toughness Test Data to establish Reference Temperature for Pressure Retaining materials of Section III, Division 1, Class 1."

In order to use the BAW-2308 methodology, an exemption is required since Appendix G to 10 CFR part 50,