

*No significant hazards consideration comments received:* No.

Dated at Rockville, Maryland, this 29th day of May, 2018.

For the Nuclear Regulatory Commission.

**Gregory F. Suber,**

*Deputy Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.*

[FR Doc. 2018-11843 Filed 6-4-18; 8:45 am]

**BILLING CODE 7590-01-P**

## **NUCLEAR REGULATORY COMMISSION**

### **Meeting of the Advisory Committee on Reactor Safeguards (ACRS) Subcommittee on APR1400**

The ACRS Subcommittee on APR1400 will hold a meeting on June 5, 2018, at 11545 Rockville Pike, Room T-2B1, Rockville, Maryland 20852.

The meeting will be open to public attendance with the exception of portions that may be closed to protect information that is proprietary pursuant to 5 U.S.C. 552b(c)(4). The agenda for the subject meeting shall be as follows:

**Tuesday, June 5, 2018, 8:30 a.m. Until 5:00 p.m.**

The Subcommittee will review the APR1400 Design Control Document and Safety Evaluation Report with No Open Items, Chapter 17 (Quality Assurance & Reliability Assurance), Chapter 19.1 (Probabilistic Risk Assessment), and Chapter 19.2 (Severe Accident Evaluation).

The Subcommittee will hear presentations by and hold discussions with the NRC staff and Korea Hydro & Nuclear Power Company regarding this matter. The Subcommittee will gather information, analyze relevant issues and facts, and formulate proposed positions and actions, as appropriate, for deliberation by the Full Committee.

Members of the public desiring to provide oral statements and/or written comments should notify the Designated Federal Official (DFO), Christopher Brown (Telephone 301-415-7111 or Email: [Christopher.Brown@nrc.gov](mailto:Christopher.Brown@nrc.gov)) five days prior to the meeting, if possible, so that appropriate arrangements can be made. Thirty-five hard copies of each presentation or handout should be provided to the DFO thirty minutes before the meeting. In addition, one electronic copy of each presentation should be emailed to the DFO one day before the meeting. If an electronic copy cannot be provided within this timeframe, presenters should provide the DFO with a CD containing each

presentation at least thirty minutes before the meeting. Electronic recordings will be permitted only during those portions of the meeting that are open to the public. Detailed procedures for the conduct of and participation in ACRS meetings were published in the **Federal Register** on October 4, 2017 (82 FR 46312). The bridgeline number for this meeting is 866-822-3032, passcode 8272423#.

Detailed meeting agendas and meeting transcripts are available on the NRC website at <http://www.nrc.gov/reading-rm/doc-collections/acrs>. Information regarding topics to be discussed, changes to the agenda, whether the meeting has been canceled or rescheduled, and the time allotted to present oral statements can be obtained from the website cited above or by contacting the identified DFO. Moreover, in view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with these references if such rescheduling would result in a major inconvenience.

If attending this meeting, please enter through the One White Flint North Building, 11555 Rockville Pike, Rockville, Maryland 20852. After registering with Security, please contact Ms. Kendra Freeland (Telephone 301-415-6207) to be escorted to the meeting room.

Dated: May 23, 2018.

**Mark L. Banks,**

*Chief, Technical Support Branch, Advisory Committee on Reactor Safeguards.*

[FR Doc. 2018-12022 Filed 6-4-18; 8:45 am]

**BILLING CODE 7590-01-P**

## **NUCLEAR REGULATORY COMMISSION**

### **Advisory Committee on Reactor Safeguards (ACRS) Meeting of the ACRS Subcommittee on Nuscale; Notice of Meeting**

The ACRS Subcommittee on NuScale will hold a meeting on June 6, 2018, at 11545 Rockville Pike, Room T-2B1, Rockville, Maryland 20852.

The meeting will be open to public attendance. The agenda for the subject meeting shall be as follows:

**Wednesday, June 6, 2018, 8:30 a.m. Until 12:00 p.m.**

The Subcommittee will review the staff's SER with open items for Chapter 8, "Electrical Systems," of the NuScale

design certification application. The Subcommittee will hear presentations by and hold discussions with the NRC staff and other interested persons regarding this matter. The Subcommittee will gather information, analyze relevant issues and facts, and formulate proposed positions and actions, as appropriate, for deliberation by the Full Committee.

Members of the public desiring to provide oral statements and/or written comments should notify the Designated Federal Official (DFO), Michael Snodderly (Telephone 301-415-2241 or Email: [Michael.Snodderly@nrc.gov](mailto:Michael.Snodderly@nrc.gov)) five days prior to the meeting, if possible, so that appropriate arrangements can be made. Thirty-five hard copies of each presentation or handout should be provided to the DFO thirty minutes before the meeting. In addition, one electronic copy of each presentation should be emailed to the DFO one day before the meeting. If an electronic copy cannot be provided within this timeframe, presenters should provide the DFO with a CD containing each presentation at least thirty minutes before the meeting. Electronic recordings will be permitted only during those portions of the meeting that are open to the public. Detailed procedures for the conduct of and participation in ACRS meetings were published in the **Federal Register** on October 4, 2017 (82 FR 46312). The bridgeline number for this meeting is 866-822-3032, passcode 8272423#.

Detailed meeting agendas and meeting transcripts are available on the NRC website at <http://www.nrc.gov/reading-rm/doc-collections/acrs>. Information regarding topics to be discussed, changes to the agenda, whether the meeting has been canceled or rescheduled, and the time allotted to present oral statements can be obtained from the website cited above or by contacting the identified DFO. Moreover, in view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with these references if such rescheduling would result in a major inconvenience.

If attending this meeting, please enter through the One White Flint North building, 11555 Rockville Pike, Rockville, Maryland. After registering with Security, please contact Mr. Theron Brown (Telephone 301-415-6702 or 301-415-8066) to be escorted to the meeting room.

Dated: May 23, 2018.

**Mark L. Banks,**

*Chief, Technical Support Branch, Advisory Committee on Reactor Safeguards.*

[FR Doc. 2018-12024 Filed 6-4-18; 8:45 am]

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## **NUCLEAR REGULATORY COMMISSION**

**[NRC-2014-0244]**

### **Guidelines for Evaluating the Effects of Light-Water Reactor Water Environments in Fatigue Analyses of Metal Components**

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Regulatory guide; issuance.

**SUMMARY:** The U.S. Nuclear Regulatory Commission (NRC) is issuing Revision 1 to Regulatory Guide (RG) 1.207, “Guidelines for Evaluating the Effects of Light-Water Reactor Water Environments in Fatigue Analyses of Metal Components.” This RG describes methods and procedures that the staff of the NRC considers acceptable for use in determining the acceptable fatigue lives of components evaluated by a cumulative usage factor calculation in accordance with the fatigue design provisions in Section III, “Rules for Construction of Nuclear Power Plant Components,” of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code to account for the effects of light-water reactor water environments.

**DATES:** Revision 1 to RG 1.207 is available on June 5, 2018.

**ADDRESSES:** Please refer to Docket ID NRC-2014-0244 when contacting the NRC about the availability of information regarding this document. You may obtain publicly-available information related to this document using any of the following methods:

- *Federal Rulemaking Website:* Go to <http://www.regulations.gov> and search for Docket ID NRC-2014-0244. Address questions about NRC dockets to Jennifer Borges; telephone: 301-287-9127; email: [Jennifer.Borges@nrc.gov](mailto:Jennifer.Borges@nrc.gov). For technical questions, contact the individuals listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *NRC’s Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly available documents online in the ADAMS Public Documents collection at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select “ADAMS Public Documents” and then

select “Begin Web-based ADAMS Search.” For problems with ADAMS, please contact the NRC’s Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov). RG 1.207, “Guidelines for Evaluating the Effects of Light-Water Reactor Water Environments in Fatigue Analyses of Metal Components,” is available in ADAMS under Accession No. ML16315A130.

- *NRC’s PDR:* You may examine and purchase copies of public documents at the NRC’s PDR, Room O1-F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

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#### **FOR FURTHER INFORMATION CONTACT:**

Robert Tregoning, Office of Nuclear Regulatory Research, telephone: 301-415-2324, email: [Robert.Tregoning@nrc.gov](mailto:Robert.Tregoning@nrc.gov) and Stephen Burton, Office of Nuclear Regulatory Research, telephone: 301-415-7000, [Stephen.Burton@nrc.gov](mailto:Stephen.Burton@nrc.gov). Both are staff of the U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

#### **SUPPLEMENTARY INFORMATION:**

##### **I. Discussion**

The NRC is issuing a revision to an existing guide in the NRC’s “Regulatory Guide” series. This series was developed to describe and make available to the public information regarding methods that are acceptable to the NRC staff for implementing specific parts of the agency’s regulations, techniques that the NRC staff uses in evaluating specific issues or postulated events, and data that the NRC staff needs in its review of applications for permits and licenses.

Revision 1 of RG 1.207 was issued with a temporary identification of Draft Regulatory Guide, DG-1309. This RG describes methods and procedures that the NRC staff considers acceptable for use in determining the acceptable fatigue lives of components evaluated by a cumulative usage factor calculation in accordance with the fatigue design provisions in Section III, “Rules for Construction of Nuclear Power Plant Components,” of the American Society of Mechanical Engineers’ Boiler and Pressure Vessel Code to account for the effects of light-water reactor water environments.

##### **II. Additional Information**

The NRC published a notice of the availability of DG-1309 in the **Federal Register** on November 24, 2014 (79 FR 69884), for a 60-day public comment

period. The public comment period closed on January 24, 2015. Public comments on DG-1309 and the NRC staff’s responses to the public comments are available in ADAMS under Accession No. ML16315A127.

##### **III. Congressional Review Act**

This RG is a rule as defined in the Congressional Review Act (5 U.S.C. 801-808). However, the Office of Management and Budget has not found it to be a major rule as defined in the Congressional Review Act.

##### **IV. Backfitting and Issue Finality**

This RG describes methods and procedures that the NRC staff considers acceptable for use in applications for license renewal and subsequent license renewal in determining the acceptable fatigue lives of components evaluated by a cumulative usage factor calculation in accordance with the fatigue design provisions in Section III, “Rules for Construction of Nuclear Power Plant Components,” of the ASME Code. This RG also supports reviews of applications for new nuclear reactor construction permits or operating licenses under part 50 of title 10 of the *Code of Federal Regulations* (10 CFR), design certifications under 10 CFR part 52, and combined licenses under 10 CFR part 52, which do not cite a standard design, in addition to renewed operating licenses under 10 CFR part 54. This RG may also be used by existing holders of combined licenses and operating licenses in accordance with their existing licensing basis and applicable regulatory requirements.

This RG does not constitute backfitting as defined in 10 CFR 50.109 (the Backfit Rule) and is not otherwise inconsistent with the issue finality provisions in 10 CFR part 52.

Applicants and potential applicants are not, with certain exceptions, protected by either the Backfit Rule or any issue finality provisions under 10 CFR part 52. Neither the Backfit Rule nor the issue finality provisions under 10 CFR part 52, with certain exclusions discussed below, were intended to apply to every NRC action that substantially changes the expectations of current and future applicants.

The exceptions to the general principle are applicable whenever a combined license applicant references a part 52 license (*i.e.*, an early site permit or a manufacturing license) and/or part 52 regulatory approval (*i.e.*, a design certification rule or design approval). The NRC staff does not, at this time, intend to impose the positions represented in the RG in a manner that is inconsistent with any issue finality