(2) the pressure spike is also localized to the wetwell area which makes up roughly 10 percent of containment, 
(3) the number of containment penetrations in this area is limited. Therefore, the current P value of 7.6 psig meets the intent of 10 CFR part 50, appendix J by bounding the peak bulk containment pressure (3.6 psig) and assuring that leakage through the primary containment does not exceed allowable leakage rate values, 
(4) the calculated peak bulk containment pressure is 3.6 psig so the Technical Specification (TS) value of 7.6 is conservative for the use of determining containment leakage, and 
(5) this request is consistent with the determination that the NRC staff has reached for other licensees under similar conditions based on the same considerations. 

The application for exemption may be examined, and/or copied for a fee, at the NRC’s Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland 20852. Publicly available records will be accessible electronically from the ADAMS Public Library component on the NRC’s Web site, http://www.nrc.gov (the Electronic Reading Room). Therefore, the NRC staff concludes that requesting exemption under the special circumstances of 10 CFR 50.12(a)(2)(ii) is appropriate and that the alternate definition of P may be used for the appendix J testing. 

Authorized by Law 

This exemption would allow Entergy to use a P value of 7.6 psig for appendix J testing at the RBS as discussed above. As stated above, 10 CFR 50.12 allows the NRC to grant exemptions from the requirements of 10 CFR part 50, appendix J. The NRC staff has determined that granting of the licensee’s proposed exemption is in accordance with the Atomic Energy Act of 1954, as amended, or the Commission’s regulations. Therefore, the exemption is authorized by law. 

No Undue Risk to Public Health and Safety 

The underlying purposes of 10 CFR part 50, appendix J are stated in section (I) “Introduction.” The purpose is to conduct tests to assure that a) leakage through the primary reactor containment does not exceed allowable leakage rate values and b) to conduct periodic surveillance of reactor containment penetrations to support proper maintenance. No new accident precursors are created because the testing is conducted at a P value calculated to be representative of peak conditions throughout containment during a design basis accident. No new accident precursors are created by use of a P of 7.6 psig instead of 9.3 psig, thus, the probability of postulated accidents is not increased. Therefore, there is no undue risk to public health and safety. 

Consistent With Common Defense and Security 

The exemption would permit exclusion of the short duration spike in wetwell pressure as P for Appendix J testing purposes. This change to the interpretation of P as defined in Appendix J has no relation to security issues. Therefore, the common defense and security is not impacted by this exemption. 

4.0 Conclusion 

Accordingly, the Commission has determined that, pursuant to 10 CFR 50.12(a), the exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with the common defense and security. Also, special circumstances are present. Therefore, the Commission hereby grants Entergy Operations, Inc., an exemption from the definition for P in 10 CFR part 50, appendix J for River Bend Station, Unit 1 and alternatively to continue to use a P value of 7.6 psig. Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will not have a significant effect on the quality of the human environment (78 FR 50454; August 19, 2013). This exemption is effective upon issuance. 

Dated at Rockville, Maryland, this 20th day of August 2013. 

For the Nuclear Regulatory Commission. 

Michele G. Evans, 
Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation. 

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) staff has determined that the inspections, tests, and analyses have been successfully completed, and that the specified acceptance criteria are met for ITAAC E.2.5.04.05.05.02, for the Vogtle Electric Generating Plant, Unit 3. 

ADDRESSES: Please refer to Docket ID NRC–2008–0252 when contacting the NRC about the availability of information regarding this document. You may access publicly-available information related to this action by the following methods: 

• Federal Rulemaking Web site: Go to http://www.regulations.gov and search for Docket ID NRC–2008–0252. Address questions about NRC dockets to Carol Gallagher; telephone: 301–287–3422; email: Carol.Gallagher@nrc.gov. For technical questions, contact the individuals listed in the FOR FURTHER INFORMATION CONTACT section of this document. 

• NRC’s Agencywide Documents Access and Management System (ADAMS): You may access publicly available documents online in the NRC Library at http://www.nrc.gov/reading-rm/adams.html. To begin the search, select “ADAMS Public Documents” and then select “Begin Web-based ADAMS search.” For problems with ADAMS, please contact the NRC’s Public Document Room (PDR) reference staff at 1–800–397–4209, 301–415–4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced in this document (if that document is available in ADAMS) is provided the first time that a document is referenced. 

• NRC’s PDR: You may examine and purchase copies of public documents at the NRC’s PDR, Room O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852. 


SUPPLEMENTARY INFORMATION: 

Licensee Notification of Completion of ITAAC 

On May 31, 2013, Southern Nuclear Operating Company, Inc. (the licensee) submitted an ITAAC closure notification (ICN) under § 52.99(c)(1) of Title 10 of the Code of Federal Regulations (10 CFR), informing the NRC that the licensee has successfully performed the required inspections, tests, and analyses for ITAAC E.2.5.04.05.05.02, and that the specified acceptance criteria are met for Vogtle.
Electric Generating Plant, Unit 3 (ADAMS Accession No. ML13154A033). This ITAAC was approved as part of the issuance of the combined license, NPF–91, for this facility.

NRC Staff Determination of Completion of ITAAC

The NRC staff has determined that the inspections, tests, and analyses have been successfully completed, and that the specified acceptance criteria are met for Vogtle Electric Generating Plant, Unit 3, ITAAC E.2.5.04.05.05.02. This notice fulfills the staff’s obligations under 10 CFR 52.99(c)(1) to publish a notice in the Federal Register of the NRC staff’s determination of the successful completion of inspections, tests, and analyses.

The documentation of the NRC staff’s determination is in the ITAAC Closure Verification Evaluation Form (VEF), dated July 10, 2013 (ADAMS Accession No. ML13191A249). The VEF is a form that represents the NRC staff’s structured process for reviewing ICNs. The ICN presents a narrative description of how the ITAAC was completed, and the NRC’s ICN review process involves a determination on whether, among other things, (1) The ICN provides sufficient information, including a summary of the methodology used to perform the ITAAC, to demonstrate that the inspections, tests, and analyses have been successfully completed; (2) the ICN provides sufficient information to demonstrate that the acceptance criteria are met; and (3) any inspections for the ITAAC have been completed and any ITAAC findings associated with the ITAAC have been closed.

The NRC staff’s determination of the successful completion of this ITAAC is based on information available at this time and is subject to the licensee’s ability to maintain the condition that the acceptance criteria are met. If new information disputes the NRC staff’s determination, this ITAAC will be reopened as necessary. The NRC staff’s determination will be used to support a subsequent finding, pursuant to 10 CFR 52.103(g), at the end of construction that all acceptance criteria in the combined license are met. The ITAAC closure process is not finalized for this ITAAC until the NRC makes an affirmative finding under 10 CFR 52.103(g). Any future updates to the status of this ITAAC will be reflected on the NRC’s Web site at http://www.nrc.gov/reactors/new-reactors/oversight/itaac.html.

Dated at Rockville, Maryland, this 22nd day of August 2013.

For the Nuclear Regulatory Commission.
Denise McGovern,
Senior Project Manager, Licensing Branch 4, Division of New Reactor Licensing, Office of New Reactors.

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BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 052–00026; NRC–2008–0252]
Inspections, Tests, Analyses, and Acceptance Criteria; Vogtle Electric Generating Plant, Unit 4

AGENCY: Nuclear Regulatory Commission.

ACTION: Determination of inspections, tests, analyses, and acceptance criteria (ITAAC) completion.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) staff has determined that the inspections, tests, and analyses have been successfully completed, and that the specified acceptance criteria are met for ITAAC E.2.5.04.05.05.02, for the Vogtle Electric Generating Plant, Unit 4.

ADDRESS: Please refer to Docket ID NRC–2008–0252 when contacting the NRC about the availability of information regarding this document. You may access publicly-available information related to this action by the following methods:
• Federal Rulemaking Web site: Go to http://www.regulations.gov and search for Docket ID NRC–2008–0252. Address questions about NRC dockets to Carol Gallagher; telephone: 301–287–3442; email: Carol.Gallagher@nrc.gov. For technical questions, contact the individuals listed in the FOR FURTHER INFORMATION CONTACT section of this document.
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• NRC’s PDR: You may examine and purchase copies of public documents at the NRC’s PDR, Room O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

FOR FURTHER INFORMATION CONTACT:

SUPPLEMENTARY INFORMATION:
Licensee Notification of Completion of ITAAC

On May 31, 2013, Southern Nuclear Operating Company, Inc. (the licensee) submitted an ITAAC closure notification (ICN) under § 52.99(c)(1) of Title 10 of the Code of Federal Regulations (10 CFR), informing the NRC that the licensee has successfully performed the required inspections, tests, and analyses for ITAAC E.2.5.04.05.05.02, and that the specified acceptance criteria are met for Vogtle Electric Generating Plant, Unit 4 (ADAMS Accession No. ML13154A032). This ITAAC was approved as part of the issuance of the combined license, NPF–92, for this facility.

NRC Staff Determination of Completion of ITAAC

The NRC staff has determined that the inspections, tests, and analyses have been successfully completed, and that the specified acceptance criteria are met for Vogtle Electric Generating Plant, Unit 4, ITAAC E.2.5.04.05.05.02. This notice fulfills the staff’s obligations under 10 CFR 52.99(c)(1) to publish a notice in the Federal Register of the NRC staff’s determination of the successful completion of inspections, tests, and analyses.

The documentation of the NRC staff’s determination is in the ITAAC Closure Verification Evaluation Form (VEF), dated July 10, 2013 (ADAMS Accession No. ML13191A249). The VEF is a form that represents the NRC staff’s structured process for reviewing ICNs. The ICN presents a narrative description of how the ITAAC was completed, and the NRC’s ICN review process involves a determination on whether, among other things, (1) The ICN provides sufficient information, including a summary of the methodology used to perform the ITAAC, to demonstrate that the inspections, tests, and analyses have been successfully completed; (2) the ICN provides sufficient information to demonstrate that the acceptance criteria are met; and (3) any inspections for the ITAAC have been completed and any ITAAC findings associated with the ITAAC have been closed.

The NRC staff’s determination of the successful completion of this ITAAC is based on information available at this time and is subject to the licensees’s ability to maintain the condition that the acceptance criteria are met. If new information disputes the NRC staff’s determination, this ITAAC will be reopened as necessary. The NRC staff’s determination will be used to support a subsequent finding, pursuant to 10 CFR 52.103(g), at the end of construction that all acceptance criteria in the combined license are met. The ITAAC closure process is not finalized for this ITAAC until the NRC makes an affirmative finding under 10 CFR 52.103(g). Any future updates to the status of this ITAAC will be reflected on the NRC’s Web site at http://www.nrc.gov/reactors/new-reactors/oversight/itaac.html.

Dated at Rockville, Maryland, this 22nd day of August 2013.