

outage. By letter dated March 16, 2013, the licensee explained the events supporting the less restrictive limitations requiring flexibility in scheduling. During the completion of the extended outage, the workload for the affected personnel will undergo a temporary but significant increase due to the various activities surrounding the significant operational events involving a fire in safety related electrical switchgear, flooding, and transitioning to Inspection Manual Chapter (IMC) 0350, "Oversight of Reactor Facilities in a Shutdown Condition due to Significant Performance and/or Operational Concerns," from being in an extended shutdown with significant performance problems. Because of these events, there has been an increase in workload prior to restart. OPPD also noted that a number of new issues have been discovered that must be tested and restored. During the extended shutdown, extensive work has been initiated to address deficiencies noted in containment building electrical penetrations, containment structural supports, and the impact of flooding hazards related to systems, structures, and components. These activities are in addition to the normal FCS startup activities involving operation and surveillance testing of primary systems and components. Ensuring a sufficient number of qualified personnel are available to support these activities is in the interest of overall public health and safety. Therefore, this exemption is consistent with the public interest.

#### 4.0 Environmental Consideration

The exemption would authorize a one-time exemption from the requirements of 10 CFR 26.205(d)(7) to allow the use of the less restrictive hour limitations described in 10 CFR 26.205(d)(4) and (d)(5). Using the standard set forth in 10 CFR 50.92 for amendments to operating licenses, the NRC staff determined that the subject exemption sought involves employment suitability requirements. The NRC has determined that this exemption involves no significant hazards considerations:

(1) The proposed exemption is administrative in nature and is limited to changing the timeframe when less restrictive hours can be worked. This does not result in any changes to the design basis requirements for the structures, systems, and components (SSCs) at FCS that function to limit the release of non-radiological effluents during and following postulated accidents. Therefore, issuance of this exemption does not increase the probability or consequences of an accident previously evaluated.

(2) The proposed exemption is administrative in nature and is limited to changing the timeframe when less restrictive hours can be worked. The proposed exemption does not make any changes to the facility or operating procedures and would not create any new accident initiators. The proposed exemption does not alter the design, function or operation of any plant equipment. Therefore, this exemption does not create the possibility of a new or different kind of accident from any accident previously evaluated.

(3) The proposed exemption is administrative in nature and is limited to changing the timeframe when less restrictive hours can be worked. The proposed exemption does not alter the design, function or operation of any plant equipment. Therefore, this exemption does not involve a significant reduction in the margin of safety.

Based on the above, the NRC concludes that the proposed exemption does not involve a significant hazards consideration under the standards set forth in 10 CFR 50.92, and accordingly, a finding of "no significant hazards consideration" is justified.

The NRC staff has also determined that the exemption involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite; that there is no significant increase in individual or cumulative occupational radiation exposure; and there is no significant increase in the potential for or consequences from a radiological accident. Furthermore, the requirement from which the licensee will be exempted involves scheduling requirements. Accordingly, the exemption meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(25). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment is required to be prepared in connection with the issuance of the exemption.

#### 5.0 Conclusion

Accordingly, the Commission has determined that pursuant to 10 CFR 26.9, "Specific exemptions," an exemption from 10 CFR 26.205(d)(7) is authorized by law and will not endanger life or property or the common defense and security, and is otherwise in the public interest.

Therefore, the Commission hereby grants OPPD a one-time, 60-day exemption from 10 CFR 26.205(d)(7) to allow the use of the work hour limitations described in 10 CFR 26.205(d)(4) and (d)(5).

This exemption is effective upon issuance. The licensee may implement

the work hour provisions of 10 CFR 26.205(d)(4) for 60 days or until the completion of the current extended outage, whichever is shorter.

Dated at Rockville, Maryland, this 11th day of June 2013.

For the Nuclear Regulatory Commission,  
**Michele G. Evans,**

*Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.*

[FR Doc. 2013-14910 Filed 6-20-13; 8:45 am]

BILLING CODE 7590-01-P

## NUCLEAR REGULATORY COMMISSION

[Docket No. 50-361; NRC-2013-0070]

### Application and Amendment to Facility Operating License Involving Proposed No Significant Hazards Consideration Determination; San Onofre Nuclear Generating Station, Unit 2

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** License amendment application; withdrawal.

**ADDRESSES:** Please refer to Docket ID NRC-2013-0070 when contacting the NRC about the availability of information regarding this document. You may access information related to this document, which the NRC possesses and is publicly available, using any of the following methods:

- *Federal Rulemaking Web site:* Go to <http://www.regulations.gov> and search for Docket ID NRC-2013-0070. Address questions about NRC dockets to Carol Gallagher; telephone: 301-492-3668; email: [Carol.Gallagher@nrc.gov](mailto:Carol.Gallagher@nrc.gov). For technical questions, contact the individual(s) listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may access publicly available documents online in the NRC Library at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "ADAMS Public Documents" and then select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov). The ADAMS accession number for each document referenced in this notice (if that document is available in ADAMS) is provided the first time that a document is referenced.

- *NRC's PDR:* You may examine and purchase copies of public documents at

the NRC's PDR, Room O1-F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

**FOR FURTHER INFORMATION CONTACT:** Brian Benney, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555; telephone: 301-415-2767; email: [Brian.Benney@nrc.gov](mailto:Brian.Benney@nrc.gov).

**SUPPLEMENTARY INFORMATION:** The U.S. Nuclear Regulatory Commission (NRC) has granted the request of Southern California Edison (the licensee) to withdraw its April 5, 2013, application (ADAMS Accession No. ML13098A043), for proposed amendment to Facility Operating License No. NPF-10 for the San Onofre Nuclear Generating Station (SONGS), Unit 2, located in San Diego County, California.

The proposed amendment would have revised the facility Technical Specification 5.5.2.11.b.1 to require that compliance with the steam generator structural integrity performance criterion (SIPC) be demonstrated up to 70% Rated Thermal Power (2406.6 megawatts thermal), and added a footnote to the Facility Operating License Condition 2.C(1) "Maximum Power Level," to restrict operation of SONGS Unit 2 to no more than 70% Rated Thermal Power for the SONGS Unit 2, Cycle 17.

The proposed amendment would have made a temporary change to the steam generator management program and the license condition for maximum power. For the duration of Unit 2, Cycle 17, the proposed amendment would have changed the terms "full range of normal operating conditions" and "normal steady state full power operation" and restricted operation to 70 percent of the maximum authorized power level. "Full range of normal operating conditions" and "normal steady state full power operation" would have been based upon the steam generators being operated under conditions associated with reactor core power levels up to 70 percent Rated Thermal Power (2406.6 megawatts thermal).

The Commission had previously issued a Notice of Consideration of Issuance of Amendment published in the **Federal Register** on April 16, 2013 (78 FR 22576). Subsequently, by letter dated June 12, 2013 (ADAMS Accession No. ML131640201), the licensee notified the NRC that SCE has permanently ceased power operation of SONGS Units 2 and 3, effective June 7, 2013. In addition, by letter dated June 13, 2013 (ADAMS Accession No. ML13165A217), the licensee notified the NRC that in light of the decision to permanently

cease power operation of SONGS Units 2 and 3, the amendment request is withdrawn.

For further details with respect to this action, see the application for amendment dated April 5, 2013, as supplemented by letters dated April 9 and May 16, 2013 (ADAMS Accession Nos. ML13100A021 and ML13137A129, respectively), and the licensee's letters dated June 12 and 13, 2013, which notified the NRC of permanent cessation of power operation at SONGS Units 2 and 3, and withdrew the application for license amendment.

Dated at Rockville, Maryland, this 14th day of June, 2013.

For the Nuclear Regulatory Commission.

**Douglas A. Broaddus,**

*Chief, SONGS Special Projects Branch, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.*

[FR Doc. 2013-14912 Filed 6-20-13; 8:45 am]

**BILLING CODE 7590-01-P**

## NUCLEAR REGULATORY COMMISSION

[NRC-2013-0105; Docket Nos. 50-325, 50-324, 50-400, 50-261]

### **Carolina Power & Light Company; Biweekly Notice; Applications and Amendments to Facility Operating Licenses Involving No Significant Hazards Considerations; Correction**

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Notice of Issuance; Correction.

**SUMMARY:** This document corrects a notice appearing in the **Federal Register** on May 28, 2013, 78 FR 31982, that inadvertently omitted the reference to Brunswick Steam Electric Plant, Unit 2, in the **Federal Register** Notice for the Carolina Power & Light Company license amendment request dated April 20, 2013. This action is necessary to include Brunswick Steam Electric Plant, Unit 2, in the **Federal Register** Notice.

**FOR FURTHER INFORMATION CONTACT:** Araceli T. Billoch Colón, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; telephone (301) 415-3302, email: [Araceli.Billoch@nrc.gov](mailto:Araceli.Billoch@nrc.gov).

**SUPPLEMENTARY INFORMATION:** On page 31982, in the first column, lines two through four, are corrected to read from "Docket No. 50-325, Brunswick Steam Electric Plant, Unit 1, Brunswick County, North Carolina" to "Docket Nos. 50-325 and 50-324, Brunswick Steam Electric Plant, Units 1 and 2, Brunswick County, North Carolina."

Dated in Rockville, Maryland, this 13th day of June 2013.

For the Nuclear Regulatory Commission.

**Araceli T. Billoch Colón,**

*Project Manager, Plant Licensing Branch II-2, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.*

[FR Doc. 2013-14879 Filed 6-20-13; 8:45 am]

**BILLING CODE 7590-01-P**

## NUCLEAR REGULATORY COMMISSION

### **Advisory Committee on Reactor Safeguards (ACRS); Meeting of the ACRS Subcommittee on Advanced Boiling Water Reactor; Notice of Meeting**

The ACRS Subcommittee on Advanced Boiling Water Reactor (ABWR) will hold a meeting on July 9, 2013, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

The meeting will be open to public attendance with the exception of a portion that may be closed to protect information that is proprietary pursuant to 5 U.S.C. 552b(c)(4). The agenda for the subject meeting shall be as follows:

**Tuesday, July 9, 2013—8:30 a.m. Until 12:00 p.m.**

The Subcommittee will review and discuss selected Chapters of the Safety Evaluation Report (SER) associated with the Combined License Application (COLA) for the South Texas Project (STP) Units 3 and 4 referencing the ABWR design. The Subcommittee will also review the proposed resolution of action items associated with the STP COLA. The Subcommittee will hear presentations by and hold discussions with the applicant, Nuclear Innovation North America (NINA), the NRC staff, and other interested persons regarding these matters. The Subcommittee will gather information, analyze relevant issues and facts, and formulate proposed positions and actions, as appropriate, for deliberation by the Full Committee.

Members of the public desiring to provide oral statements and/or written comments should notify the Designated Federal Official (DFO), Maitri Banerjee (Telephone 301-415-3718 or Email: [Maitri.Banerjee@nrc.gov](mailto:Maitri.Banerjee@nrc.gov)) five days prior to the meeting, if possible, so that appropriate arrangements can be made. Thirty-five hard copies of each presentation or handout should be provided to the DFO thirty minutes before the meeting. In addition, one electronic copy of each presentation should be emailed to the DFO one day before the meeting. If an electronic copy