

site and its dose-mitigating engineered safety features remain acceptable with respect to the radiological consequences of postulated DBAs, since the calculated doses to members of the public meet the exposure guideline values specified in 10 CFR 50.67 and General Design

Criteria 19 in Appendix A of 10 CFR Part 50. If the NRC should approve LAR 241, then the proposed EPU will not have a significant human health impact with respect to radiological consequences of DBAs.

Radiological Impacts Summary

As discussed above, the proposed EPU would not result in any significant radiological impacts. Table 2 summarizes the radiological environmental impacts of the proposed EPU at the PBNP.

TABLE 2—SUMMARY OF RADIOLOGICAL ENVIRONMENTAL IMPACTS

| | |
|---|---|
| Radioactive Gaseous Effluents | Amount of additional radioactive gaseous effluents generated would be handled by the existing system. |
| Radioactive Liquid Effluents | Amount of additional radioactive liquid effluents generated would be handled by the existing system. |
| Occupational Radiation Doses | Occupational doses would continue to be maintained within NRC limits. |
| Offsite Radiation Doses | Radiation doses to members of the public would remain below NRC and EPA radiation protection standards. |
| Radioactive Solid Waste | Amount of additional radioactive solid waste generated would be handled by the existing system. |
| Spent Nuclear Fuel | Amount of additional spent nuclear fuel would be handled by the existing system. |
| Postulated Design- Basis Accident Doses | Calculated doses for postulated design-basis accidents would remain within NRC limits. |

Alternatives to the Proposed Action

As an alternative to the proposed action, the NRC staff considered denial of the proposed EPU (*i.e.*, the “no-action” alternative). Denial of the application would result in no change in the current environmental impacts. However, if the EPU were not approved for the PBNP, other agencies and electric power organizations may be required to pursue other means, such as fossil fuel or alternative fuel power generation, to provide electric generation capacity to offset future demand. Construction and operation of such a fossil-fueled or alternative-fueled plant may create impacts in air quality, land use, and waste management significantly greater than those identified for the proposed EPU at the PBNP. Furthermore, the proposed EPU does not involve environmental impacts that are significantly different from those originally identified in the PBNP Unit 1 or Unit 2 FESs, and the SEIS-23.

Alternative Use of Resources

The action does not involve the use of any resources than those previously considered in the FES.

Agencies and Persons Consulted

In accordance with its stated policy, on April 6, 2011, the NRC staff consulted with the State of Wisconsin official regarding the environmental impact of the proposed action. The State official had no comments. Comments were received from the Wisconsin PSC and incorporated into the EA. The Wisconsin PSC has no objections to the projects as proposed.

Finding of No Significant Impact

On the basis of the details provided in the EA, the NRC concludes that the

proposed action of implementing the PBNP EPU will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee’s application dated April 7, 2009, and supplements dated May 13, 2010, and July 15, 2010 (on environmental issues).

Documents may be examined, and/or copied for a fee, at the NRC’s Public Document Room (PDR), located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland 20852. Publicly available records are available online in the NRC Library at <http://www.nrc.gov/reading-rm/adams.html>. From this page, the public can gain entry into ADAMS, which provides text and image files of the NRC’s public documents. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC PDR Reference staff at 1-800-397-4209, or 301-415-4737, or send an e-mail to pdr.resource@nrc.gov.

Dated at Rockville, Maryland, this 13th day of April 2011.

For the Nuclear Regulatory Commission.

Terry A. Beltz,

Senior Project Manager, Plant Licensing Branch III-1, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. 2011-9835 Filed 4-22-11; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards (ACRS), Meeting of the ACRS Subcommittee on Reliability and Probabilistic Risk Assessment; Notice of Meeting

The ACRS Subcommittee on Reliability and Probabilistic Risk Assessment (PRA) will hold a meeting on May 11, 2011, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Monday, May 11, 2011—1 p.m. until 5 p.m.

The Subcommittee will hear a briefing on the plan and schedule for developing a level 3 PRA. The Subcommittee will hear presentations by and hold discussions with the NRC staff and other interested persons regarding this matter. The Subcommittee will gather information, analyze relevant issues and facts, and formulate proposed positions and actions, as appropriate, for deliberation by the Full Committee.

Members of the public desiring to provide oral statements and/or written comments should notify the Designated Federal Official (DFO), John Lai (Telephone 301-415-5197 or E-mail: John.Lai@nrc.gov) five days prior to the meeting, if possible, so that appropriate arrangements can be made. Thirty-five hard copies of each presentation or handout should be provided to the DFO thirty minutes before the meeting. In addition, one electronic copy of each presentation should be emailed to the DFO one day before the meeting. If an electronic copy cannot be provided

within this timeframe, presenters should provide the DFO with a CD containing each presentation at least thirty minutes before the meeting. Electronic recordings will be permitted only during those portions of the meeting that are open to the public. Detailed procedures for the conduct of and participation in ACRS meetings were published in the **Federal Register** on October 21, 2010 (75 FR 65038–65039).

Detailed meeting agendas and meeting transcripts are available on the NRC Web site at <http://www.nrc.gov/reading-rm/doc-collections/acrs>. Information regarding topics to be discussed, changes to the agenda, whether the meeting has been canceled or rescheduled, and the time allotted to present oral statements can be obtained from the Web site cited above or by contacting the identified DFO.

Moreover, in view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with these references if such rescheduling would result in a major inconvenience.

If attending this meeting, please contact Ms. Jessie Delgado (Telephone 301–415–7360) to be escorted to the meeting room.

Dated: April 19, 2011.

Yoira Diaz-Sanabria,

*Acting Chief, Reactor Safety Branch B,
Advisory Committee on Reactor Safeguards.*

[FR Doc. 2011–9918 Filed 4–22–11; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards (ACRS); Meeting of the ACRS Subcommittee on U.S. Evolutionary Power Reactor; Notice of Meeting

The ACRS Subcommittee on U.S. Evolutionary Power Reactor (U.S. EPR) will hold a meeting on May 11, 2011, Room T–2B1, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Wednesday, May 11, 2011—8:30 a.m. until 12 p.m.

The Subcommittee will review Chapter 18 of the Safety Evaluation Report (SER) with open items associated with the review of the U.S. EPR Design Control Document (DCD). The

Subcommittee will hear presentations by and hold discussions with AREVA, the NRC staff, and other interested persons regarding this matter. The Subcommittee will gather information, analyze relevant issues and facts, and formulate proposed positions and actions, as appropriate, for deliberation by the Full Committee.

Members of the public desiring to provide oral statements and/or written comments should notify the Designated Federal Official (DFO), Derek Widmayer (Telephone 301–415–7366 or E-mail: Derek.Widmayer@nrc.gov) five days prior to the meeting, if possible, so that appropriate arrangements can be made. Thirty-five hard copies of each presentation or handout should be provided to the DFO thirty minutes before the meeting. In addition, one electronic copy of each presentation should be emailed to the DFO one day before the meeting. If an electronic copy cannot be provided within this timeframe, presenters should provide the DFO with a CD containing each presentation at least thirty minutes before the meeting. Electronic recordings will be permitted only during those portions of the meeting that are open to the public. Detailed procedures for the conduct of and participation in ACRS meetings were published in the **Federal Register** on October 21, 2010, (75 FR 65038–65039).

Detailed meeting agendas and meeting transcripts are available on the NRC Web site at <http://www.nrc.gov/reading-rm/doc-collections/acrs>. Information regarding topics to be discussed, changes to the agenda, whether the meeting has been canceled or rescheduled, and the time allotted to present oral statements can be obtained from the Web site cited above or by contacting the identified DFO. Moreover, in view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with these references if such rescheduling would result in a major inconvenience.

Dated: April 19, 2011.

Yoira Diaz-Sanabria,

*Acting Chief, Reactor Safety Branch B,
Advisory Committee on Reactor Safeguards.*

[FR Doc. 2011–9920 Filed 4–22–11; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 72–8; NRC–2011–0085]

Calvert Cliffs Nuclear Power Plant, LLC Independent Spent Fuel Storage Installation; Notice of Docketing, Notice of Proposed Action, and Notice of Opportunity for a Hearing for Renewal of Special Nuclear Materials License No.–2505

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of license renewal, and opportunity to request a hearing.

DATES: A request for hearing and/or petition for leave to intervene must be filed by June 24, 2011.

FOR FURTHER INFORMATION CONTACT: John M. Goshen, Project Manager, Licensing Branch, Division of Spent Fuel Storage and Transportation, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555–0001. Telephone: 301–492–3325; fax number: 301–492–3348; e-mail: john.goshen@nrc.gov.

SUPPLEMENTARY INFORMATION:

I. Introduction

The U.S. Nuclear Regulatory Commission (NRC or Commission) is considering an application dated September 17, 2010, as supplemented February 10, and March 9, 2011, Calvert Cliffs Nuclear Power Plant (CCNPP), LLC, for the renewal of its Special Nuclear Material (SNM) License No.–2505, under the provisions of 10 CFR Part 72, for the receipt, possession, storage and transfer of spent fuel, reactor-related Greater than Class C (GTCC) waste and other radioactive materials associated with spent fuel storage at the CCNPP site-specific Independent Spent Fuel Storage Installation (ISFSI), located near Lusby, Maryland. If granted, the renewed license will authorize the applicant to continue to store spent fuel in a dry cask storage system at the applicant's CCNPP ISFSI. Pursuant to the provisions of 10 CFR 72.42, the renewal term of the license for the ISFSI would be twenty (20) years; however, the applicant has submitted an exemption request with the license renewal application, which, if granted, would allow the license to be renewed for 40 years. On February 16, 2011, (76 FR 8872) the revised 10 CFR 72.230(b) rule was published changing the renewal term to a period not to exceed 40 years to be effective May 17, 2011. The exemption request is no longer required. The renewal