

originally noticed, and did not change the staff's original proposed no significant hazards consideration determination as published in the **Federal Register**.

The Commission's related evaluation of the amendment is contained in a safety evaluation dated March 27, 2009.

No significant hazards consideration comments received: No.

Pacific Gas and Electric Company, Docket Nos. 50-275 and 50-323, Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, San Luis Obispo County, California

Date of application for amendments: April 3, 2008, as supplemented by letters dated June 20, October 1, November 6, and December 16, 2008.

Brief description of amendments: The amendments revised Technical Specification (TS) 3.7.5, "Auxiliary Feedwater (AFW) System," to remove Surveillance Requirement (SR) 3.7.5.6, and revised TS 3.7.6, "Condensate Storage Tank (CST) and Fire Water Storage Tank (FWST)," to remove the FWST level requirements, revise the CST level requirements, and revise TS 3.7.6 to be consistent with the NUREG-1431, "Standard Technical Specifications (STS)." Specifically, these changes reflect design changes made to the CSTs and are necessary to support the on-line refurbishment of the FWST and replacement of the recirculation piping for the fire water pumps. The design changes to the CSTs are intended to eliminate the reliance on the FWST for additional seismically-qualified feedwater supply and thus, make the existing TS requirements for the FWST unnecessary.

Date of issuance: March 30, 2009.

Effective date: As of its date of issuance and shall be implemented within 60 days from the date of issuance.

Amendment Nos.: Unit 1-204; Unit 2-205.

Facility Operating License Nos. DPR-80 and DPR-82: The amendments revised the Facility Operating Licenses and Technical Specifications.

Date of initial notice in Federal Register: July 29, 2008 (78 FR 43956). The supplemental letters dated June 20, October 1, November 6, and December 16, 2008, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination as published in the **Federal Register**.

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated March 30, 2009.

No significant hazards consideration comments received: No.

Tennessee Valley Authority, Docket No. 50-390, Watts Bar Nuclear Plant (WBN), Unit 1, Rhea County, Tennessee

Date of application for amendment: September 18, 2008.

Brief description of amendment: The amendment revised WBN Unit 1 Technical Specification 3.8.7, "Inverters—Operating." The amendment revised the requirement to two inverters for each of the four channels.

Date of issuance: March 24, 2009.

Effective date: As of the date of issuance and shall be implemented within 240 days of issuance.

Amendment No.: 76.

Facility Operating License No. NPF-90: Amendment revises the Technical Specification 3.8.7 and Updated Final Safety Analysis Report.

Date of initial notice in Federal Register: November 4, 2008 (73 FR 65697).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 24, 2009.

No significant hazards consideration comments received: No.

Wolf Creek Nuclear Operating Corporation, Docket No. 50-482, Wolf Creek Generating Station, Coffey County, Kansas

Date of amendment request: March 14, 2007, as supplemented by letters dated April 18, May 9, June 15, August 31, September 12 and 20, October 16, November 16, two letters dated December 14, and December 18, 2007; two letters dated January 18, January 31, February 26 and 28, March 14, April 26, May 14, June 19, and July 31, 2008; and January 16 and 29, and February 17 and 27, 2009.

Brief description of amendment: The amendment revised the licensing basis for the Main Steam and Feedwater Isolation System (MSFIS) controls to incorporate field programmable gate array technology. Other related changes requested in the March 14, 2007, application were previously approved in Amendment No. 174, dated August 28, 2007, Amendment No. 175, dated March 3, 2008, Amendment No. 176, dated March 21, 2008, and Amendment No. 177, dated April 3, 2008.

Date of issuance: March 31, 2009.

Effective date: Effective as of date of issuance and shall be implemented before entry into Mode 3 in the restart from Refueling Outage 17.

Amendment No.: 181.

Renewed Facility Operating License No. NPF-42. The amendment revised the Operating License.

Date of initial notice in Federal Register: June 19, 2007 (72 FR 33785). The supplemental letters dated April 18, May 9, June 15, August 31, September 12 and 20, October 16, November 16, two letters dated December 14, and December 18, 2007; two letters dated January 18, January 31, February 26 and 28, March 14, April 26, May 14, June 19, and July 31, 2008; and January 16 and 29, and February 17 and 27, 2009, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination as published in the **Federal Register**.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 31, 2009.

No significant hazards consideration comments received: No.

Dated at Rockville, Maryland, this 10th day of April 2009.

For the Nuclear Regulatory Commission.

Joseph G. Giitter,

Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. E9-8832 Filed 4-20-09; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[NRC-2009-0176]

Draft Regulatory Guide: Issuance, Availability

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of Issuance and Availability of Draft Regulatory Guide, DG-1214.

FOR FURTHER INFORMATION CONTACT: Dan Frumkin, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, telephone (301) 415-2280, e-mail Dan.Frumkin@nrc.gov, or, R. A. Jervey, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, telephone: (301) 251-7407, e-mail to raj@nrc.gov.

SUPPLEMENTARY INFORMATION:

I. Introduction

The U.S. Nuclear Regulatory Commission (NRC) is issuing for public comment a draft guide in the agency's "Regulatory Guide" series. This series was developed to describe and make available to the public such information

as methods that are acceptable to the NRC staff for implementing specific parts of the NRC's regulations, techniques that the staff uses in evaluating specific problems or postulated accidents, and data that the staff needs in its review of applications for permits and licenses.

The draft regulatory guide (DG), titled, "Fire Protection for Nuclear Power Plants," is temporarily identified by its task number, DG-1214, which should be mentioned in all related correspondence. DG-1214 is proposed Revision 2 of Regulatory Guide 1.189.

The primary objectives of fire protection programs (FPPs) at U.S. nuclear plants are to minimize both the probability of occurrence and the consequences of fire. To meet these objectives, the FPPs for operating nuclear power plants are designed to provide reasonable assurance, through defense in depth, that a fire will not prevent the necessary safe-shutdown functions from being performed and that radioactive releases to the environment in the event of a fire will be minimized.

The regulatory framework that the NRC has established for nuclear plant FPPs consists of a number of regulations and supporting guidelines, including, but not limited to, Title 10 of the *Code of Federal Regulations*, Part 50, "Domestic Licensing of Production and Utilization Facilities," (10 CFR Part 50), Appendix A, "General Design Criteria for Nuclear Power Plants," General Design Criterion (GDC) 3, "Fire Protection;" 10 CFR 50.48, "Fire Protection;" Appendix R, "Fire Protection Program for Nuclear Power Facilities Operating Prior to January 1, 1979," to 10 CFR Part 50; regulatory guides; generic communications (e.g., generic letters [GLs], regulatory issue summaries [RISs], bulletins, and information notices [INs]); NUREG-series reports, including NUREG-0800, "Standard Review Plan [SRP] for the Review of Safety Analysis Reports for Nuclear Power Plants;" and industry standards. Since not all of the fire protection regulations promulgated by the NRC apply to all plants, this guide does not categorize them as regulations. Licensees should refer to their plant-specific licensing bases to determine the applicability of a specific regulation to a specific plant.

The NRC staff developed this guide to provide a comprehensive fire protection guidance document and to identify the scope and depth of fire protection that the staff would consider acceptable for nuclear power plants. The original issue of this guide addressed only plants operating as of January 1, 2001. Revision 1 of the document added guidance for

new reactor designs and incorporated the guidance previously included in Branch Technical Position (BTP) SPLB 9.5-1, "Guidelines for Fire Protection for Nuclear Power Plants (formerly BTP CMEB 9.5-1)." DG-1214 incorporates guidance related to analysis of safe-shutdown capabilities as found in regulatory position 5.3.

II. Further Information

The NRC staff is soliciting comments on DG-1214. Comments may be accompanied by relevant information or supporting data and should mention DG-1214 in the subject line. Comments submitted in writing or in electronic form will be made available to the public in their entirety through the NRC's Agencywide Documents Access and Management System (ADAMS).

Personal information will not be removed from your comments. You may submit comments by any of the following methods:

1. Mail comments to: Rulemaking, Directives, and Editing Branch, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

2. E-mail comments to: nrcprep.resource@nrc.gov.

Requests for technical information about DG-1214 may be directed to the NRC contact, Dan Frumkin at (301) 415-2280 or e-mail to Dan.Frumkin@nrc.gov.

Comments would be most helpful if received by May 29, 2009. Comments received after that date will be considered if it is practical to do so, but the NRC is able to ensure consideration only for comments received on or before this date. Although a time limit is given, comments and suggestions in connection with items for inclusion in guides currently being developed or improvements in all published guides are encouraged at any time.

Electronic copies of DG-1214 are available through the NRC's public Web site under Draft Regulatory Guides in the "Regulatory Guides" collection of the NRC's Electronic Reading Room at <http://www.nrc.gov/reading-rm/doc-collections/>. Electronic copies are also available in ADAMS (<http://www.nrc.gov/reading-rm/adams.html>), under Accession No. ML090070453.

In addition, regulatory guides are available for inspection at the NRC's Public Document Room (PDR), which is located at 11555 Rockville Pike, Rockville, Maryland. The PDR's mailing address is USNRC PDR, Washington, DC 20555-0001. The PDR can also be reached by telephone at (301) 415-4737 or (800) 397-4205, by fax at (301) 415-3548, and by e-mail to pdr.resource@nrc.gov.

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Dated at Rockville, Maryland, this 7th day of April 2009.

For the Nuclear Regulatory Commission.

Andrea D. Valentin,

Chief, Regulatory Guide Development Branch, Division of Engineering, Office of Nuclear Regulatory Research.

[FR Doc. E9-9099 Filed 4-20-09; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards

In accordance with the purposes of Sections 29 and 182b of the Atomic Energy Act (42 U.S.C. 2039, 2232b), the Advisory Committee on Reactor Safeguards (ACRS) will hold a meeting on May 7-9, 2009, 11545 Rockville Pike, Rockville, Maryland. The date of this meeting was previously published in the **Federal Register** on Monday, October 6, 2008 (73 FR 58268-58269).

Thursday, May 7, 2009, Conference Room T-2b3, Two White Flint North, Rockville, Maryland

8:30 a.m.-8:35 a.m.: Opening Remarks by the ACRS Chairman

(Open)—The ACRS Chairman will make opening remarks regarding the conduct of the meeting.

8:35 a.m.-10:30 a.m.: Proposed Rule on Risk-Informed Changes to Loss-of-Coolant Accident Technical Requirements (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed rule on a voluntary risk-informed alternative to the current requirements of emergency core cooling systems, and related matters.

10:45 a.m.-12:15 p.m.: Proposed Resolution of Generic Safety Issue (GSI)-163, "Multiple Steam Generator Tube Leakage" (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed resolution of GSI-163 regarding multiple steam generator tube leakage, and related matters.

1:15 p.m.-2:45 p.m.: Draft Final Regulatory Guide 1.214, "Response Procedures for Potential or Actual Aircraft Attacks" (Open/Closed)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the draft final Regulatory Guide 1.214 and related matters. [**Note:**