Place: Call will originate from the National Science Foundation, 4201 Wilson Boulevard, Arlington, Virginia.

Type of Meeting: Closed.

Contact Person: Jennifer Richards, Committee Executive Secretary, National Science Board Office, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230. Telephone: (703) 292–7000. E-mail: *jlrichar@nsf.gov.*

Purpose of Meeting: To provide advice and recommendations in the selection of the NSB Public Service Award recipient.

Agenda: Discussion of candidates for the NSB Public Service Award as part of the selection process.

Reason for Meeting Closure: The candidate nominations being reviewed include information of a personal nature where public disclosure would constitute clearly unwarranted invasions of personal privacy. These matters are exempt from open meeting and public attendance under 5 U.S.C. 55b(c)(6).

Dated: November 10, 2008.

Suzanne H. Plimpton,

Reports Clearance Officer, National Science Foundation.

[FR Doc. E8–27132 Filed 11–13–08; 8:45 am]

BILLING CODE 7555–01–P

NUCLEAR REGULATORY COMMISSION

NUREG/CR–XXXX, ''Modeling a Digital Feedwater Control System Using Traditional Probabilistic Risk Assessment Methods''; Draft Report for Comment

AGENCY: U.S. Nuclear Regulatory Commission.

ACTION: Notice of availability for public comment.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is conducting research to support development of regulatory guidance for using risk information related to digital systems in the licensing actions of nuclear power plants (NPPs). The objective of this research is to identify and develop methods, analytical tools, and regulatory guidance to support (1) using information on the risks of digital systems in NPP licensing decisions and (2) including models of digital systems into NPP probabilistic risk assessments (PRAs).

As part of this research, NRC is sponsoring a project on the use of traditional PRA methods to develop and quantitatively assess reliability models of digital systems. The initial tasks of this project, including preparatory work for developing reliability models of an example system, are addressed in NUREG/CR–6962, ''Traditional Probabilistic Risk Assessment Methods for Digital Systems'' (to be published shortly). The application of the selected traditional methods to the example system is documented in draft NUREG/CR–XXXX, ''Modeling a Digital Feedwater Control System Using Traditional Probabilistic Risk Assessment Methods.'' This notice announces the availability of the draft NUREG/CR for public comment.

DATES: Please submit comments on NUREG/CR–XXXX, ''Modeling a Digital Feedwater Control System Using Traditional Probabilistic Risk Assessment Methods,'' by December 29, 2008. Comments received after this date will be considered if practical to do so, but the NRC staff is able to ensure consideration only for those comments received on or before this date.

ADDRESSES: NUREG/CR–XXXX, ''Modeling a Digital Feedwater Control System Using Traditional Probabilistic Risk Assessment Methods,'' is available for inspection and copying for a fee at NRC's Public Document Room (PDR), Public File Area O–1F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland. Publicly available documents created or received at NRC after November 1, 1999, are available electronically at NRC's Electronic Reading Room at *http://www.nrc.gov/NRC/ADAMS/index.html.* From this site, the public can gain entry into NRC's Agencywide Document Access and Management System (ADAMS), which provides text and image files of NRC's public documents. The ADAMS Accession Numbers for NUREG/CR–XXXX, ''Modeling a Digital Feedwater Control System Using Traditional Probabilistic Risk Assessment Methods,'' are ML082800062 (main report) and ML082800063 (appendices). If you do not have access to ADAMS or have problems accessing the documents located in ADAMS, contact the NRC PDR Reference staff at 1–800–397–4209, (301) 415–4737, or by e-mail to *pdr@nrc.gov.*

This document also will be posted on NRC's public Web site at: *http://www.nrc.gov/about-nrc/regulatory/research/digital/tech-reference.html#one.*

Please submit comments to Chief, Rulemaking, Directives and Editing Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001. You also may deliver comments to 11545 Rockville Pike, Rockville, MD, between 7:30 a.m. and 4:30 p.m. on Federal workdays, or by e-mail to: *nrcrep@nrc.gov.*

FOR FURTHER INFORMATION CONTACT: Alan Kuritzky, Office of Nuclear Regulatory Research, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, telephone (301) 415–6255, e-mail: *Alan.Kuritzky@nrc.gov.*

Dated at Rockville, Maryland this 6th day of November, 2008.

For the U.S. Nuclear Regulatory Commission.

Christiana Lui,

Director, Division of Risk Analysis, Office of Nuclear Regulatory Research.

[FR Doc. E8–27100 Filed 11–13–08; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–366]

Southern Nuclear Operating Company, Inc.; Edwin I. Hatch Nuclear Plant, Unit No. 2; Exemption

1.0 Background

The Southern Nuclear Operating Company, Inc. (SNC, the licensee) is the holder of the Renewed Facility Operating License No. NPF–5 which authorizes operation of the Edwin I. Hatch Nuclear Plant, Unit No. 2 (HNP–2). The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the U.S. Nuclear Regulatory Commission (NRC or the Commission) now or hereafter in effect.

The facility consists of a boiling-water reactor located in Appling County in Georgia.

2.0 Request/Action

Pursuant to Title 10 of the Code of Federal Regulations (10 CFR), Section 50.12, ''Specific Exemptions'', SNC has, by letters dated March 21, May 2, August 8 and September 22, 2008, requested an exemption from the fuel cladding material requirements in 10 CFR 50.46, ''Acceptance Criteria for Emergency Core Cooling Systems for Light-Water Nuclear Power Reactors,'' and Appendix K to 10 CFR part 50, ''ECCS Evaluation Models,'' (Appendix K). The regulation in 10 CFR 50.46 contains acceptance criteria for emergency core cooling system (ECCS) for reactors fueled with zircaloy or ZIRLO™ cladding. In addition, Appendix K requires that the Baker-Just equation be used to predict the rates of energy release, hydrogen concentration, and cladding oxidation from the metal-water reaction. The exemption request relates solely to the specific types of cladding material specified in these regulations. As written, the regulations