

**NATIONAL SCIENCE FOUNDATION****Advisory Committee for Social, Behavioral, and Economic Sciences; Notice of Meeting**

In accordance with the Federal Advisory Committee Act (Pub. L. 92-463, as amended), the National Science Foundation announces the following meeting:

*Name:* Advisory Committee for Social, Behavioral, and Economic Sciences (#1171).

*Date/Time:* November 20, 2008; 8:30 a.m. to 5:30 p.m.; November 21, 2008; 8:30 a.m. to 11:45 a.m.

*Place:* National Science Foundation, 4201 Wilson Boulevard, Stafford I, Third Floor—Room 375, Arlington, VA 22230.

*Type of Meeting:* Open.

*Contact Person:* Ms. Lisa Jones, Office of the Assistant Director, Directorate for Social, Behavioral, and Economic Sciences, National Science Foundation, 4201 Wilson Boulevard, Room 905, Arlington, Virginia 22230, 703-292-8700.

*Summary of Minutes:* May be obtained from contact person listed above.

*Purpose of Meeting:* To provide advice and recommendation to the National Science Foundation on major goals and policies pertaining to Social, Behavioral and Economic Sciences Directorate programs and activities.

**Agenda***Thursday*

Updates and Discussions on Continuing Activities

CyberInfrastructure, Cyber-enabled Discovery and Innovation, and SBE Sciences

- Data: Massive data sets, data preservation, etc.
- Virtual organizations: For the SBE sciences and as a subject of SBE study
- Complexity sciences in SBE

Facilitating Transformative and Interdisciplinary Research  
Science of Broadening Participation:  
Workshop Report

Update on SRS activities

*Friday*

Updates and Discussion on Continuing Activities

Upcoming Committees of Visitors

- SLC Program
- SRS Division
- BCS Division

Discussion with the NSF Director  
Planning for FY 2010 and Beyond

Dated: October 29, 2008.

**Susanne Bolton,**

*Committee Management Officer.*

[FR Doc. E8-26112 Filed 10-31-08; 8:45 am]

**BILLING CODE 7555-01-P**

**NATIONAL SCIENCE FOUNDATION****Business and Operations Advisory Committee; Notice of Meeting**

In accordance with Federal Advisory Committee Act (Pub. L. 92-463, as amended), the National Science Foundation announces the following meeting:

*Name:* Business and Operations Advisory Committee (9556).

*Date/Time:* November 19, 2008; 1 p.m. to 5:30 p.m. (EST); November 20, 2008; 8 a.m. to 12 p.m. (EST).

*Place:* National Science Foundation, 4201 Wilson Boulevard, Stafford II, Room 555.

*Type of Meeting:* Open.

*Contact Person:* Joan Miller, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230, (703) 292-8200.

*Purpose of Meeting:* To provide advice concerning issues related to the oversight, integrity, development and enhancement of NSF's business operations.

**Agenda***November 19, 2008*

PM: Welcome/Introductions; OIRM/CIO/BFA Updates; NSF Policy Updates; Science of Science Innovation Policy (SciSIP); BFA Human Capital Strategic Plan Update; B&O Advisory Committee: Strategic Directions and Planning.

*November 20, 2008*

AM: Committee Discussion: Prepare for Meeting with NSF Director; Discussion with Director; Continue Strategy and Planning Discussion & Prepare Recommendations; Committee Discussion/Wrap-Up.

Dated: October 29, 2008.

**Susanne Bolton,**

*Committee Management Officer.*

[FR Doc. E8-26111 Filed 10-31-08; 8:45 am]

**BILLING CODE 7555-01-P**

**NUCLEAR REGULATORY COMMISSION****Draft Regulatory Guide: Granting Extension of Comment Period**

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Notice of Granting of Request To Extend the Comment Period of Draft Regulatory Guide (DG)-1200, "An Approach for Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities."

**FOR FURTHER INFORMATION CONTACT:**

Mary Drouin, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, telephone: (301) 415-6675 or e-mail to [Mary.Drouin@nrc.gov](mailto:Mary.Drouin@nrc.gov).

**SUPPLEMENTARY INFORMATION:****I. Introduction**

The U.S. Nuclear Regulatory Commission (NRC) issued for public comment DG-1200, which was published in the **Federal Register**, 73 FR 35170, on June 20, 2008. DG-1200 is proposed Revision 2 of Regulatory Guide 1.200 in the agency's Regulatory Guide Series. This series has been developed to describe and make available to the public such information as methods that are acceptable to the NRC staff for implementing specific parts of the NRC's regulations, techniques that the staff uses in evaluating specific problems or postulated accidents, and data that the staff needs in its review of applications for permits and licenses.

**II. Further Information**

The NRC staff requested receipt of comments on DG-1200 by September 24, 2008, (including any implementation schedule) and its associated regulatory analysis or value/impact statement. By this action, the NRC staff is extending the comment period until December 31, 2008. Comments received after December 31, 2008, would be considered if practical to do so, but the NRC is able to ensure consideration only for comments received on or before this date. Although a time limit is given, comments and suggestions in connection with items for inclusion in guides currently being developed or improvements in all published guides are encouraged at any time.

**III. Request To Extend the Comment Period***Basis for the Request*

The NRC received the following extension request:

In a joint letter, the American Society of Mechanical Engineers (ASME) and the American Nuclear Society (ANS) requested that the public review and comment period on Draft Guide 1200 (proposed Revision 2 of Regulatory Guide [RG] 1.200) be extended to the end of December 2008 with a subsequent publication date of Revision 2 to RG 1.200 ("An Approach for Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities") extended to the end of March 2009. The extension will allow Addendum A to the "Standard for Level 1/Large Early Release Frequency Probabilistic Risk Assessment for Nuclear Power Plant Applications" (ASME/ANS RA-S-2008) to be endorsed in Revision 2 to RG 1.200.

*Response to Request*

The request for an extension to the comment period is approved until December 31, 2008.

We agree that Addendum A is a needed improvement to the ASME/ANS standard RA-S-2008 and would result in a more effective and efficient implementation of both the standard and RG 1.200. Consequently, we are extending the public review and comment period to December 31, 2008, and the final publication of Revision 2 to RG 1.200 to March 31, 2009.

ASME and ANS have cooperated worked to develop this PRA standard in support of NRC's PRA quality initiative/plan in support of risk-informed regulation (SECY-04-0118, "Plan for the Implementation of the Commission's Phased Approach to Probabilistic Risk Assessment Quality," dated July 13, 2004). This plan identified the various technical guidance documents (*e.g.*, standards) needed to support the various risk-informed activities.

Requests for technical information about DG-1200 may be directed to the NRC contact, Mary Drouin at (301) 415-6675 or e-mail to [Mary.Drouin@nrc.gov](mailto:Mary.Drouin@nrc.gov).

Electronic copies of DG-1200 are available through the NRC's public Web site under Draft Regulatory Guides in the "Regulatory Guides" collection of the NRC's Electronic Reading Room at <http://www.nrc.gov/reading-rm/doc-collections/>. Electronic copies are also available in ADAMS (<http://www.nrc.gov/reading-rm/adams.html>), under Accession No. ML081200566.

In addition, regulatory guides are available for inspection at the NRC's Public Document Room (PDR), which is located at 11555 Rockville Pike, Rockville, Maryland. The PDR's mailing address is USNRC PDR, Washington, DC 20555-0001. The PDR can also be reached by telephone at (301) 415-4737 or (800) 397-4205, by fax at (301) 415-3548, and by e-mail to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov).

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Dated at Rockville, Maryland, this 29 day of October, 2008.

For The Nuclear Regulatory Commission.

**Andrea D. Valentin,**

*Chief, Regulatory Guide Development Branch, Division of Engineering, Office of Nuclear Regulatory Research.*

[FR Doc. E8-26214 Filed 10-31-08; 8:45 am]

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**NUCLEAR REGULATORY COMMISSION**

[Docket No. 50-366]

**Edwin I. Hatch Nuclear Plant, Unit No. 2; Southern Nuclear Operating Company, Inc.; Operating License No. NPF-5; Environmental Assessment and Finding of No Significant Impact**

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from Title 10 of the Code of Federal Regulations (10 CFR) Part 50, Section 46, and Appendix K to Part 50, for Facility Operating License No. NPF-5, issued to Southern Nuclear Operating Company (the licensee), for operation of the Edwin I. Hatch Nuclear Plant, Unit 2 located in Appling County, Georgia. Therefore, as required by 10 CFR 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

**Environmental Assessment**

*Identification of the Proposed Action*

The proposed action would allow the use of Ziron fuel cladding. The proposed action is in accordance with the licensee's application dated March 21, 2008, as supplemented May 2, August 8, and September 22, 2008.

*The Need for the Proposed Action*

The proposed action would allow a small number of lead test assemblies (LTAs) that will include some fuel rods manufactured with a cladding material, called GNF-Ziron, which is similar in composition to Zircaloy-2, but contains a slightly higher iron content than specified in ASTM B350. Irradiation of LTAs with GNF-Ziron fuel rods will enable SNC to acquire in-reactor operating experience with this material. Pursuant to 10 CFR 50.12, "Specific Exemptions," the licensee has requested an exemption to 10 CFR 50.46, "acceptance criteria for emergency core cooling systems for light-water nuclear power reactors," that requires, among other items, that "each boiling or pressurized light-water nuclear power reactor fueled with uranium oxide pellets within cylindrical zircaloy or ZIRLO cladding, must be provided with an emergency core cooling system (ECCS) that must be designed so that its calculated cooling performance following postulated loss-of-coolant accidents conforms to the criteria set forth in paragraph (b) of this section." Appendix K to 10 CFR Part 50, "ECCS Evaluation Models," requires, among other items, that the rate of energy release, hydrogen generation, and cladding oxidation from the metal/water

reaction shall be calculated using the Baker-Just equation. The regulations at 10 CFR 50.46 and 10 CFR Part 50, Appendix K, make no provisions for use of fuel rods clad in a material other than zircaloy or ZIRLO. The proposed action would allow the licensee to irradiate a small number of LTAs using fuel rods clad with Ziron alloy in Hatch, Unit 2. Since the material specifications of the Ziron alloy differ from the specification for zircaloy or ZIRLO, a plant-specific exemption is required to support the use of the eight assemblies.

*Environmental Impacts of the Proposed Action*

The NRC has completed its safety evaluation of the proposed action and concludes that application of 10 CFR 50.46, and Appendix K to 10 CFR Part 50, is not necessary for the licensee to achieve its underlying purposes.

The details of the NRC staff safety evaluation will be provided in the exemption that will be issued as part of the letter to the licensee approving the exemption to the regulation.

The staff has concluded that such a change would not adversely affect plant safety, and would have no adverse effect on the probability of any accident. For accidents that involve damage or melting of the fuel in the reactor core, the fuel rod integrity of GNF-Ziron clad fuel has been shown to be similar to zircaloy clad fuel; therefore, the probability of an accident will not be affected. For accidents in which the core remains intact, the use of GNF-Ziron cladding will not have a significant effect on the mix of fission products that could be released in the event of a serious accident; thus, the previously analyzed accident dose consequences remain bounding. Regulatory limits on radiological effluent releases are independent of the type of fuel cladding used. The requirements of 10 CFR 50.36a, Appendix I to 10 CFR Part 50, and 40 CFR Part 190, as well as the plant's Technical Specifications ensure that the release of radioactive gaseous, liquid, and solid waste to unrestricted areas are kept to "as low as reasonably achievable" (ALARA) levels. The licensee's radioactive waste processing system will collect, control, process to reduce the amount of radioactivity, and discharge the waste in accordance with regulatory limits. Therefore, the staff concluded that during routine operations, there will be no significant increase of radiological effluents released into the environment as a result of the proposed exemption request. No significant increase in the allowable individual occupational radiation