

assurance of consideration cannot be given to comments received after this date.

Nathan J. Frey, Office of Information and Regulatory Affairs (3150-0193), NEOB-10202, Office of Management and Budget, Washington, DC 20503.

Comments can also be e-mailed to Nathan_J._Frey@omb.eop.gov or submitted by telephone at (202) 395-7345.

The NRC Clearance Officer is Margaret A. Janney, (301) 415-7245.

Dated at Rockville, Maryland, this 8th day of July, 2008.

For the Nuclear Regulatory Commission.

Tremaine Donnell,

Acting NRC Clearance Officer, Office of Information Services.

[FR Doc. E8-15926 Filed 7-11-08; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 52-018-COL, 52-019-COL; ASLBP No. 08-865-03-COL-BD01]

Duke Energy Carolinas, LLC; Establishment of Atomic Safety and Licensing Board

Pursuant to delegation by the Commission dated December 29, 1972, published in the *Federal Register*, 37 FR 28,710 (1972), and the Commission's regulations, see 10 CFR 2.104, 2.300, 2.303, 2.309, 2.311, 2.318, and 2.321, notice is hereby given that an Atomic Safety and Licensing Board (Board) is being established to preside over the following proceeding:

Duke Energy Carolinas, LLC

(William States Lee III Nuclear Station, Units 1 and 2)

This proceeding concerns a Petition to Intervene and Request for Hearing submitted by the Blue Ridge Environmental Defense League, and a request to participate in any hearing by the South Carolina Office of Regulatory Staff, which were submitted in response to an April 28, 2008 Notice of Hearing and Opportunity To Petition for Leave To Intervene on a Combined License for William States Lee III Units 1 and 2 (73 FR 22,978). The Petition to Intervene and Request for Hearing challenges the application filed by Duke Energy Carolinas, LLC, pursuant to Subpart C of 10 CFR Part 52 for a combined license for William States Lee III Nuclear Station, Units 1 and 2, which would be located in Cherokee County, South Carolina.

The Board is comprised of the following administrative judges:

Paul S. Ryerson, Chair, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001;
Nicholas G. Trikouros, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001;
Dr. William H. Murphy, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

All correspondence, documents, and other materials shall be filed in accordance with the NRC E-Filing rule, which the NRC promulgated in August 2007 (72 FR 49,139).

Issued at Rockville, Maryland, this 8th day of July 2008.

Anthony J. Baratta,

Associate Chief Administrative Judge—Technical, Atomic Safety and Licensing Board Panel.

[FR Doc. E8-16008 Filed 7-11-08; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-424 and 50-425]

Southern Nuclear Operating Company, Inc., Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, City of Dalton, GA, Vogtle Electric Generating Plant, Units 1 and 2; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing and Order Imposing Procedures for Access to Sensitive Unclassified Non-Safeguards Information

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-68 and NPF-81 issued to the Southern Nuclear Operating Company, Inc. (the licensee), acting for itself, Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the owners), for operation of the Vogtle Electric Generating Plant (VEGP), Units 1 and 2 (VEGP Units 1 and 2) located in Wayne County, Georgia.

This amendment application proposes a one-time steam generator (SG) tubing eddy current inspection interval revision to the VEGP Units 1 and 2, Technical Specifications (TSs) 5.5.9, "Steam Generator (SG) Program," to incorporate an interim alternate repair

criterion in the provisions for SG tube repair criteria during the Unit 2 inspection performed in Refueling Outage 13 and subsequent operating cycle. This amendment application requests approval of an interim alternate repair criterion (IARC) that requires full-length inspection of the tubes within the tubesheet but does not require plugging tubes if any axial or circumferential cracking observed in the region greater than 17 inches below the top of the tubesheet (TTS) is less than a value sufficient to permit the remaining circumferential ligament to transmit the limiting axial loads. This amendment application is required to preclude unnecessary plugging while still maintaining structural and leakage integrity.

This amendment application includes SUNSI (proprietary information). Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in Title 10 of the Code of Federal Regulations (10 CFR), Section 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

(1) Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

Of the various accidents previously evaluated, the proposed changes only affect the steam generator tube rupture (SGTR) event evaluation and the postulated steam line break (SLB), locked rotor and control rod ejection accident evaluations. Loss-of-coolant accident (LOCA) conditions cause a compressive axial load to act on the tube. Therefore, since the LOCA tends to force the tube into the tubesheet rather than pull it out, it is not a factor in this licensing amendment request. Another faulted load consideration is a safe shutdown earthquake (SSE); however, the seismic analysis of Model F steam generators has shown that axial loading of the tubes is negligible during an SSE.