

**Matters To Be Considered**

*Week of November 20, 2006*

There are no meetings scheduled for the Week of November 20, 2006.

*Week of November 27, 2006—Tentative*

Thursday, November 30, 2006

12:55 p.m. Affirmation Session (Public Meeting) (Tentative), a. Hydro Resources, Inc. (Crownpoint, NM) Intervenor's Petition for Review of LBP-06-19 (Final Partial Initial Decision—NEPA Issues) (Tentative).

*Week of December 4, 2006—Tentative*

Thursday, December 7, 2006

9:30 a.m. Discussion of Management Issues (Closed—Ex. 2).

*Week of December 11, 2006—Tentative*

Monday, December 11, 2006

1:30 p.m. Briefing on Status of Decommissioning Activities (Public Meeting) (Contact: Keith McConnell, 301-415-7295).

This meeting will be Webcast live at the Web address—<http://www.nrc.gov>.

Tuesday, December 12, 2006

9:30 a.m. Briefing on Threat Environment Assessment (Closed—Ex. 1)

1:30 p.m. Discussion of Security Issues (Closed—Ex. 1 & 3).

Wednesday, December 13, 2006

9:30 a.m. Briefing on Status of Equal Employment Opportunity (EEO) Programs (Public Meeting) (Contact: Barbara Williams, 301-415-7388).

This meeting will be Webcast live at the Web address—<http://www.nrc.gov>.

Thursday, December 14, 2006

9:30 a.m. Meeting with Advisory Committee on Nuclear Waste (ACNW) (Public Meeting) (Contact: John Larkins, 301-415-7360).

This meeting will be webcast live at the Web address—<http://www.nrc.gov>.

*Week of December 18, 2006—Tentative*

There are no meetings scheduled for the Week of December 18, 2006.

*Week of December 25, 2006—Tentative*

There are no meetings scheduled for the Week of December 25, 2006.

\*The schedule for Commission meetings is subject to change on short notice. To verify the status of meetings call (recording)—301-415-1292. Contact person for more information: Michelle Schroll, 301-415-1662.

The NRC Commission Meeting Schedule can be found on the Internet at: <http://www.nrc.gov/what-we-do/policy-making/schedule.html>.

The NRC provides reasonable accommodation to individuals with disabilities where appropriate. If you need a reasonable accommodation to participate in these public meetings, or need this meeting notice or the transcript or other information from the public meetings in another format (e.g. braille, large print), please notify the NRC's Disability Program Coordinator, Deborah Chan, at 301-415-7041, TDD: 301-415-2100, or by e-mail at [DLG@nrc.gov](mailto:DLG@nrc.gov). Determinations on requests for reasonable accommodation will be made on a case-by-case basis.

This notice is distributed by mail to several hundred subscribers; if you no longer wish to receive it, or would like to be added to the distribution, please contact the Office of the Secretary, Washington, DC 20555 (301-415-1969). In addition, distribution of this meeting notice over the Internet system is available. If you are interested in receiving this Commission meeting schedule electronically, please send an electronic message to [dkw@nrc.gov](mailto:dkw@nrc.gov).

Dated: November 14, 2006.

**R. Michelle Schroll,**

*Office of the Secretary.*

[FR Doc. 06-9292 Filed 11-16-06; 10:17 am]

**BILLING CODE 7590-01-M**

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## **NUCLEAR REGULATORY COMMISSION**

**Notice of Opportunity To Comment on Model Safety Evaluation and Model License Amendment Request on Technical Specification Improvement Regarding Deletion of E Bar Definition and Revision to Reactor Coolant System Specific Activity Technical Specification; Babcock and Wilcox Pressurized Water Reactors, Westinghouse Pressurized Water Reactors, Combustion Engineering Pressurized Water Reactors Using the Consolidated Line Item Improvement Process**

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Request for comment.

**SUMMARY:** Notice is hereby given that the staff of the U.S. Nuclear Regulatory Commission (NRC) has prepared a model license amendment request (LAR), model safety evaluation (SE), and model proposed no significant hazards consideration (NSHC) determination related to deletion of E Bar (average disintegration energy,  $\bar{E}$ ) definition and revision to reactor coolant system (RCS) specific activity technical specification. This request revises the RCS specific activity specification for pressurized

water reactors to utilize a new indicator, Dose Equivalent Xenon-133 instead of the current indicator known as E Bar.

The purpose of these models is to permit the NRC to efficiently process amendments to incorporate these changes into plant-specific Technical Specifications (TS) for Babcock and Wilcox, Westinghouse, and Combustion Engineering Pressurized Water Reactors (PWRs). Licensees of nuclear power reactors to which the models apply can request amendments conforming to the models. In such a request, a licensee should confirm the applicability of the model LAR, model SE and NSHC determination to its plant. The NRC staff is requesting comments on the model LAR, model SE and NSHC determination for referencing in license amendment applications.

**DATES:** The comment period expires 30 days from the date of this publication. Comments received after this date will be considered if it is practical to do so, but the Commission is able to ensure consideration only for comments received on or before this date.

**ADDRESSES:** Comments may be submitted either electronically or via U.S. mail.

*Submit written comments to:* Chief, Rulemaking, Directives, and Editing Branch, Division of Administrative Services, Office of Administration, Mail Stop: T-6 D59, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

*Hand deliver comments to:* 11545 Rockville Pike, Rockville, Maryland, between 7:45 a.m. and 4:15 p.m. on Federal workdays.

*Submit comments by electronic mail to:* [CLIP@nrc.gov](mailto:CLIP@nrc.gov).

Copies of comments received may be examined at the NRC's Public Document Room, One White Flint North, Public File Area O1-F21, 11555 Rockville Pike (first floor), Rockville, Maryland.

**FOR FURTHER INFORMATION CONTACT:** Trent Wertz, Mail Stop: O-12H2, Division of Inspection and Regional Support, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, telephone (301) 415-1568.

### **SUPPLEMENTARY INFORMATION:**

#### **Background**

Regulatory Issue Summary 2000-06, "Consolidated Line Item Improvement Process (CLIP) for Adopting Standard Technical Specifications Changes for Power Reactors," was issued on March 20, 2000. The CLIP is intended to improve the efficiency and transparency of NRC licensing processes. This is accomplished by processing proposed

changes to the Standard Technical Specifications (STS) in a manner that supports subsequent license amendment applications. The CLIIP includes an opportunity for the public to comment on proposed changes to the STS following a preliminary assessment by the NRC staff and finding that the change will likely be offered for adoption by licensees. This notice is soliciting comment on a proposed change to the STS that deletes the E Bar definition and revises the RCS specific activity technical specification of the Babcock and Wilcox PWR STS, Revision 3 of NUREG-1430, Westinghouse PWR STS Revision 3 NUREG-1431, and Combustion Engineering PWR STS Revision 3 NUREG-1432. The CLIIP directs the NRC staff to evaluate any comments received for a proposed change to the STS and to either reconsider the change or proceed with announcing the availability of the change for proposed adoption by licensees. Those licensees opting to apply for the subject change to TSs are responsible for reviewing the staff's evaluation, referencing the applicable technical justifications, and providing any necessary plant-specific information. Following the public comment period, the model LAR and model SE will be finalized, and posted on the NRC webpage. Each amendment application made in response to the notice of availability will be processed and noticed in accordance with applicable NRC rules and procedures.

This notice involves replacement of the current PWR TS 3.4.16 limits on RCS gross specific activity with a new limit on RCS noble gas specific activity. The noble gas specific activity limit would be based on a new dose equivalent Xe-133 (DEX) definition that would replace the current E-Bar average disintegration energy definition. In addition, the current dose equivalent I-131 (DEI) definition would be revised to allow the use of additional thyroid dose conversion factors (DCFs). By letter dated September 13, 2005, the Technical Specifications Task Force (TSTF) proposed these changes for incorporation into the STS as TSTF-490, Revision 0.

#### Applicability

These proposed changes will revise the definition of DOSE EQUIVALENT I-131, delete the definition of "E-Bar," AVERAGE DISINTEGRATION ENERGY, add a new definition for DOSE EQUIVALENT XE-133, and revise LCO 3.4.16 for Babcock and Wilcox, Westinghouse, and Combustion Engineering PWRs, STS NUREGs 1430, 1431, and 1432.

To efficiently process the incoming license amendment applications, the NRC staff requests that each licensee applying for the changes addressed by TSTF-490, Revision 0, using the CLIIP submit an LAR that adheres to the following model. Any variations from the model LAR should be explained in the licensee's submittal. Variations from the approach recommended in this notice may require additional review by the NRC staff, and may increase the time and resources needed for the review. Significant variations from the approach, or inclusion of additional amendment requests, may result in staff rejection of the CLIIP adoption request. Instead, licensees desiring significant variations and/or additional changes should submit a non-CLIIP LAR that does not request to adopt TSTF-490 via CLIIP.

#### Public Notices

This notice requests comments from interested members of the public within 30 days of the date of this publication. Following the NRC staff's evaluation of comments received as a result of this notice, the NRC staff may reconsider the proposed change or may proceed with announcing the availability of the change in a subsequent notice (perhaps with some changes to the model LAR, model SE or model NSHC determination as a result of public comments). If the NRC staff announces the availability of the change, licensees wishing to adopt the change will submit an application in accordance with applicable rules and other regulatory requirements. The NRC staff will, in turn, issue for each application a notice of consideration of issuance of amendment to facility operating license(s), a proposed NSHC determination, and an opportunity for a hearing. A notice of issuance of an amendment to operating license will announce the revised requirements for each plant that applies for and receives the requested change.

Dated at Rockville, Maryland this 14th day of November, 2006.

For the Nuclear Regulatory Commission.

**Timothy J. Kobetz,**

*Chief, Technical Specifications Branch,  
Division of Inspection and Regional Support,  
Office of Nuclear Reactor Regulation.*

For inclusion on the technical specification web page. The following example of an application was prepared by the NRC staff to facilitate the adoption of Technical Specifications task force (TSTF) Traveler TSTF-490, Revision 0 "deletion of E-bar definition and revision to RCS Specific Activity Tech Spec." The model provides the expected level of detail and content for an application to adopt TSTF-490, Revision 0. Licensees remain responsible for ensuring

that their actual application fulfills their administrative requirements as well as NRC regulations.

U.S. Nuclear Regulatory Commission,  
Document Control Desk,  
Washington, DC 20555.

SUBJECT: Plant Name, Docket No. 50-[xxx,]  
Re: Application for Technical Specification Improvement to Adopt TSTF-490, Revision 0, "deletion of E-bar definition and revision to RCS Specific Activity Tech Spec"

Dear Sir or Madam: In accordance with the provisions of Section 50.90 of Title 10 of the Code of Federal Regulations (10 CFR), [LICENSEE] is submitting a request for an amendment to the technical specifications (TS) for [PLANT NAME, UNIT NOS.]. The proposed changes would replace the current pressurized water reactor (PWR) Technical Specification (TS) 3.4.16 limits on reactor coolant system (RCS) gross specific activity with a new limit on RCS noble gas specific activity. The noble gas specific activity limit would be based on a new dose equivalent Xe-133 (DEX) definition that would replace the current E-Bar (E) average disintegration energy definition. In addition, the current dose equivalent I-131 (DEI) definition would be revised to allow the use of additional thyroid dose conversion factors (DCFs).

The changes are consistent with NRC-approved Industry Technical Specification Task Force (TSTF) Standard Technical Specification Change Traveler, TSTF-490, Revision 0, "Deletion of E-Bar Definition and Revision to RCS Specific Activity Tech Spec." The availability of this TS improvement was announced in the **Federal Register** on [DATE] ([ ]FR[ ]) as part of the Consolidated Line Item Improvement Process (CLIIP).

Enclosure 1 provides a description and assessment of the proposed changes, as well as confirmation of applicability. Enclosure 2 provides the existing TS pages and TS Bases marked-up to show the proposed changes. Enclosure 3 provides final TS pages and TS Bases pages.

[LICENSEE] requests approval of the proposed license amendment by [DATE], with the amendment being implemented [BY DATE OR WITHIN X DAYS]. In accordance with 10 CFR 50.91, a copy of this application, with enclosures, is being provided to the designated [STATE] Official.

I declare under penalty of perjury under the laws of the United States of America that I am authorized by [LICENSEE] to make this request and that the foregoing is true and correct. [Note that request may be notarized in lieu of using this oath or affirmation statement]. If you should have any questions regarding this submittal, please contact [ ].

Sincerely,  
Name, Title

Enclosures:

1. Description and Assessment of Proposed Changes
2. [Proposed Technical Specification Changes and Technical Specification Bases Changes]
3. [Final Technical Specification and Bases pages]

cc: NRR Project Manager, Regional Office,  
Resident Inspector, State Contact, ITSB  
Branch Chief.

## Enclosure 1—Description and Assessment of Proposed Changes

### 1.0 Description

This letter is a request to amend Operating License(s) [LICENSE NUMBER(S)] for [PLANT/UNIT NAME(S)].

The proposed changes would replace the current limits on primary coolant gross specific activity with limits on primary coolant noble gas activity. The noble gas activity would be based on DOSE EQUIVALENT XE-133 and would take into account only the noble gas activity in the primary coolant.

Technical Specification Task Force (TSTF) change traveler TSTF-490, Revision 0, "Deletion of E Bar Definition and Revision to RCS Specific Activity Tech Spec" was announced for availability in the **Federal Register** on [DATE] as part of the consolidated line item improvement process (CLIP).

### 2.0 Proposed Changes

Consistent with NRC-approved TSTF-490, Revision 0, the proposed TS changes include:

- Revised definition of DOSE EQUIVALENT I-131
  - Deletion of the definition of "E-Bar, AVERAGE DISINTEGRATION ENERGY EQUIVALENT XE-133
  - Adding a new definition for DOSE EQUIVALENT XE-133
  - Revised LCO 3.4.16, "RCS Specific Activity" to delete references to gross specific activity, and reference limits on DOSE EQUIVALENT I-131 and DOSE EQUIVALENT XE-133, and delete Figure 3.4.16-1, "Reactor Coolant DOSE EQUIVALENT I-131 Specific Activity Limit versus Percent of RATED THERMAL POWER."
  - Revised Applicability of LCO 3.4.16 to indicate the LCO is applicable in MODES 1,2,3, and 4
  - [Modified ACTIONS Table as follows:
    - A. Condition A is modified to delete the reference to Figure 3.4.16-1, and define an upper limit that is applicable at all power levels.
    - B. NUREG-1430 and NUREG-1432 ACTIONS are reordered, moving Condition C to Condition B.
    - C. Condition B (was Condition C in NUREG-1430 and NUREG 1432) is modified to provide a Condition and Required Action for DOSE EQUIVALENT XE-133 instead of gross specific activity. The Completion Time is changed from 6 hours to 48 hours. A Note stating the applicability of LCO 3.0.4.c is added, consistent with the Note to Required Action A.1.
    - D. Condition C (was Condition B in NUREG-1430 and NUREG-1432) is modified based on the changes to Conditions A and B and to reflect the change in the LCO Applicability]
      - Revised SR 3.4.16.1 to verify the limit for DOSE EQUIVALENT XE-133. A Note is added, consistent with SR 3.4.16.2 to allow entry into MODES 4,3, and 2 prior to performance of the SR.

- Deleted SR 3.4.16.3

### 3.0 Background

The background for this application is as stated in the model SE in NRC's Notice of Availability published on [DATE] ([ ] FR [ ]), the NRC Notice for Comment published on [DATE] ([ ] FR [ ]), and TSTF-490, Revision 0.

### 4.0 Technical Analysis

[LICENSEE] has reviewed References 1 and 2, and the model SE published on [DATE] ([ ] FR [ ]) as part of the CLIP Notice for Availability. [LICENSEE] has applied the methodology in Reference 1 to develop the proposed TS changes. [LICENSEE] has also concluded that the justifications presented in TSTF-490, Revision 0 and the model SE prepared by the NRC staff are applicable to [PLANT, UNIT NOS.], and justify this amendment for the incorporation of the changes to the [PLANT] TS.

### 5.0 Regulatory Analysis

A description of this proposed change and its relationship to applicable regulatory requirements and guidance was provided in the NRC Notice of Availability published on [DATE] ([ ] FR [ ]), the NRC Notice for Comment published on [DATE] ([ ] FR [ ]), and TSTF-490, Revision 0.

### 6.0 No Significant Hazards Consideration

[LICENSEE] has reviewed the proposed no significant hazards consideration determination published in the **Federal Register** on [DATE] ([ ] FR [ ]) as part of the CLIP. [LICENSEE] has concluded that the proposed determination presented in the notice is applicable to [PLANT] and the determination is hereby incorporated by reference to satisfy the requirements of 10 CFR 50.91(a).

### 7.0 Environmental Evaluation

[LICENSEE] has reviewed the environmental consideration included in the model SE published in the **Federal Register** on [DATE] ([ ] FR [ ]) as part of the CLIP. [LICENSEE] has concluded that the staff's findings presented therein are applicable to [PLANT] and the determination is hereby incorporated by reference for this application.

### 8.0 References

1. Federal Register Notices:
  - Notice for Comment published on [DATE] ([ ] FR [ ])
  - Notice of Availability published on [DATE] ([ ] FR [ ])

## MODEL SAFETY EVALUATION

### U.S. Nuclear Regulatory Commission

#### Office of Nuclear Reactor Regulation

#### Technical Specification Task Force TSTF-490, Revision 0

#### "Deletion of E-Bar Definition and Revision to RCS Specific Activity Tech Spec"

### 1.0 INTRODUCTION

By letter dated [\_, 20\_], [LICENSEE] (the licensee) proposed changes to the technical specifications (TS) for [PLANT NAME]. The

requested changes are the adoption of TSTF-490, Revision 0, "Deletion of E Bar Definition and Revision to RCS Specific Activity Tech Spec" to the Pressurized Water Reactor (PWR) Standard Technical Specifications (STS), which was proposed by the Technical Specifications Task Force (TSTF) by letter on September 13, 2005. This TSTF involves changes to NUREG-1430, NUREG-1431, and NUREG-1432 Section 3.4.16 limits on reactor coolant system (RCS) gross specific activity with a new limit on RCS noble gas specific activity. The noble gas specific activity limit would be based on a new dose equivalent Xe-133 (DEX) definition that would replace the current E-Bar average disintegration energy definition. In addition, the current dose equivalent I-131 (DEI) definition would be revised to allow the use of additional thyroid dose conversion factors (DCFs).

## 2.0 REGULATORY EVALUATION

The Nuclear Regulatory Commission (NRC) staff evaluated the impact of the proposed changes as they relate to the radiological consequences of affected design basis accidents (DBAs) that use the reactor coolant system (RCS) inventory as the source term. The source term assumed in radiological analyses should be based on the activity associated with the projected fuel damage or the maximum TS RCS values, whichever maximizes the radiological consequences. The limits on RCS specific activity ensure that the offsite doses are appropriately limited for accidents that are based on releases from the RCS with no significant amount of fuel damage.

The Steam Generator Tube Rupture (SGTR) accident and the Main Steam Line Break (MSLB) accident, with a few exceptions, do not result in fuel damage and therefore the radiological consequence analyses are based on the release of primary coolant activity at maximum TS limits. For accidents that result in fuel damage, the additional dose contribution from the initial activity in the RCS is not normally evaluated and is considered to be insignificant in relation to the dose resulting from the release of fission products from the damaged fuel.

[For licensees that incorporate the source term as defined in Technical Information Document (TID) 14844, AEC, 1962, "Calculation of Distance Factors for Power and Test Reactors Sites," in their dose consequence analyses, the staff uses the regulatory guidance provided in NUREG-0800, "Standard Review Plan (SRP) for the Review of Safety Analysis Reports for Nuclear Power Plants," Section 15.1.5, "Steam System Piping Failures Inside and Outside of Containment (PWR)," Appendix A, "Radiological Consequences of Main Steam Line Failures Outside Containment," Revision 2, for the evaluation of MSLB accident analyses and NUREG-0800, SRP Section 15.6.3, "Radiological Consequences of Steam Generator Tube Failure (PWR)," Revision 2, for evaluating SGTR accidents analyses. In addition, the staff uses the guidance from RG 1.195, "Methods and Assumptions for Evaluating Radiological Consequences of Design Basis Accidents at Light Water Nuclear Power Reactors," May 2003, for those licensees that chose to use its

guidance for dose consequence analyses using the TID 14844 source term.]

[For licensees using the alternative source term (AST) in their dose consequence analyses, the staff uses the regulatory guidance provided in NUREG-0800, SRP Section 15.0.1, "Radiological Consequence Analyses Using Alternative Source Terms," Revision 0, July 2000, and the methodology and assumptions stated in Regulatory Guide (RG) 1.183, "Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors", July 2000.]

The applicable dose criteria for the evaluation of DBAs depends on the source term incorporated in the dose consequence analyses. [For licensees using the TID 14844 source term, the maximum dose criteria to the whole body and the thyroid that an individual at the exclusion area boundary (EAB) can receive for the first 2 hours following an accident, and at the low population zone (LPZ) outer boundary for the duration of the radiological release, are specified in Title 10 of the Code of Federal Regulations (10 CFR) Part 100.11. These criteria are 25 roentgen equivalent man (rem) total whole body dose and 300 rem thyroid dose from iodine exposure. The accident dose criteria in 10 CFR 100.11 is supplemented by accident specific dose acceptance criteria in SRP 15.1.5, Appendix A, SRP 15.6.3 or Table 4 of RG 1.195, "Methods and Assumptions for Evaluating Radiological Consequences of Design Basis Accidents at Light Water Nuclear Power Reactors," May 2003.]

[For control room dose consequence analyses that use the TID 14844 source term, the regulatory requirement for which the NRC staff bases its acceptance is General Design Criterion (GDC) 19 of Appendix A to 10 CFR Part 50, "Control Room". GDC 19 requires that adequate radiation protection be provided to permit access and occupancy of the control room under accident conditions without personnel receiving radiation exposures in excess of 5 rem whole body, or its equivalent to any part of the body, for the duration of the accident. NUREG-0800, SRP Section 6.4, "Control Room Habitability System," Revision 2, July 1981, provides guidelines defining the dose equivalency of 5 rem whole body as 30 rem for both the thyroid and skin dose. For licensees adopting the guidance from RG 1.196, "Control Room Habitability at Light Water Nuclear Power Reactors," May 2003, Section C.4.5 of RG 1.195, May 2003, states that in lieu of the dose equivalency guidelines from Section 6.4 of NUREG-0800, the 10 CFR 20.1201 annual organ dose limit of 50 rem can be used for both the thyroid and skin dose equivalent of 5 rem whole body.]

[Licensees using the AST are evaluated against the dose criteria specified in 10 CFR Part 50.67. The off-site dose criteria are 25 rem total effective dose equivalent (TEDE) at the EAB for any 2-hour period following the onset of the postulated fission product release and 25 rem TEDE at the outer boundary of the LPZ for the duration of the postulated fission product release. In addition, 10 CFR Part 50.67 requires that adequate radiation protection be provided to permit access and occupancy of the control

room under accident conditions without personnel receiving radiation exposures in excess of 5 rem TEDE for the duration of the accident.]

### 3.0 TECHNICAL EVALUATION

#### 3.1 Technical Evaluation of TSTF-490 RCS TS Changes

##### 3.1.1 Revision to the Definition of DEI

The list of acceptable DCFs for use in the determination of DEI include the following:

- Table III of TID-14844, AEC, 1962, "Calculation of Distance Factors for Power and Test Reactor Sites."
- Table E-7 of Regulatory Guide 1.109, Revision 1, NRC, 1977.
- ICRP 30,1979, page 192-212, Table titled "Committed Dose Equivalent in Target Organs or Tissues per Intake of Unit Activity."
- Table 2.1 of EPA Federal Guidance Report No. 11, 1988, "Limiting Values of Radionuclide Intake and Air Concentration and Dose Conversion Factors for Inhalation, Submersion, and Ingestion."

Note: It is incumbent on the licensee to ensure that the DCFs used in the determination of DEI are consistent with the applicable dose consequence analyses.

##### 3.1.2 Deletion of the Definition of E-Bar and the Addition of a New Definition for DE Xe-133

The determination of DEX will be performed in a similar manner to that currently used in determining DEI, except that the calculation of DEX is based on the acute dose to the whole body and considers the noble gases Kr-85m, Kr-87, Kr-88, Xe-133m, Xe-133, Xe-135m, Xe-135, and Xe-138 which are significant in terms of contribution to whole body dose. Some noble gas isotopes are not included due to low concentration, short half life, or small dose conversion factor. The calculation of DEX would use either the average gamma disintegration energies for the nuclides or the effective dose conversion factors from Table III.1 of EPA Federal Guidance Report No.12, "External Exposure to Radionuclides in Air, Water, and Soil", 1993. Using this approach, the limit on the amount of noble gas activity in the primary coolant would not fluctuate with variations in the calculated values of E-Bar. If a specified noble gas nuclide is not detected, the new definition states that it should be assumed the nuclide is present at the minimum detectable activity. This will result in a conservative calculation of DEX.

When E-Bar is determined using a design basis approach in which it is assumed that 1.0% of the power is being generated by fuel rods having cladding defects and it is also assumed that there is no removal of fission gases from the letdown flow, the value of E-Bar is dominated by Xe-133. The other nuclides have relatively small contributions. However, during normal plant operation there are typically only a small amount of fuel clad defects and the radioactive nuclide inventory can become dominated by tritium and corrosion and or activation products, resulting in the determination of a value of E-Bar that is very different than would be calculated using the design basis approach. Because of this difference the accident dose

analyses become disconnected from plant operation and the limiting condition for operation (LCO) becomes essentially meaningless. It also results in a TS limit that can vary during operation as different values for E-Bar are determined.

This change will implement a LCO that is consistent with the whole body radiological consequence analyses which are sensitive to the noble gas activity in the primary coolant but not to other, non-gaseous activity currently captured in the E-Bar definition. SR 3.4.16.1 specifies the limit for primary coolant gross specific activity as 100/E-Bar Ci/gm. The current E-Bar definition includes radioisotopes that decay by the emission of both gamma and beta radiation. The current Condition B of LCO 3.4.16 would rarely, if ever, be met for exceeding 100/E-Bar since the calculated value is very high (the denominator is very low) if beta emitters such as tritium (H-3) are included in the determination, as required by the E-Bar definition.

TS Section 1.1 definition for E-AVERAGE DISINTEGRATION ENERGY (E-Bar) is deleted and replaced with a new definition for DEX which states:

"DOSE EQUIVALENT XE-133 shall be that concentration of Xe-133 (microcuries per gram) that alone would produce the same acute dose to the whole body as the combined activities of noble gas nuclides Kr-85m, Kr-85, Kr-87, Kr-88, Xe-131m, Xe-133m, Xe-133, Xe-135m, Xe-135, and Xe-138 actually present. If a specific noble gas nuclide is not detected, it should be assumed to be present at the minimum detectable activity. The determination of DOSE EQUIVALENT XE-133 shall be performed using effective dose conversion factors for air submersion listed in Table III.1 of EPA Federal Guidance Report No. 12, or the average gamma disintegration energies as provided in ICRP Publication 38, "Radionuclide Transformations" or similar source."

The change incorporating the newly defined quantity DEX is acceptable from a radiological dose perspective since it will result in an LCO that more closely relates the non-iodine RCS activity limits to the dose consequence analyses which form their bases. NOTE: IT IS INCUMBENT ON THE LICENSEE TO ENSURE THAT THE DCFs USED IN THE DETERMINATION OF DEI AND THE NEWLY DEFINED DEX ARE CONSISTENT WITH THE DCFs USED IN THE APPLICABLE DOSE CONSEQUENCE ANALYSIS.

##### 3.1.3 Revision of LCO 3.4.16, "RCS Specific Activity"

LCO 3.4.16 is modified to specify that iodine specific activity in terms of DEI and noble gas specific activity in terms of DEX shall be within limits. Currently the limiting indicators are not explicitly identified in the LCO, but are instead defined in current Condition C and Surveillance Requirement (SR) 3.4.16.1 for gross non-iodine specific activity and in current Condition A and SR 3.4.16.2 for iodine specific activity.

The new LCO states "RCS DOSE EQUIVALENT I-131 and DOSE EQUIVALENT XE-133 specific activity shall be within limits." NOTE: IT IS INCUMBENT

ON THE LICENSEE TO ENSURE THAT THE SITE SPECIFIC LIMITS FOR BOTH DEI AND DEX ARE CONSISTENT WITH THE CURRENT SGTR AND MSLB RADIOLOGICAL CONSEQUENCE ANALYSES.

TS 3.4.16 Required Action A.1 is revised to remove the reference to Figure 3.4.16-1 "Reactor Coolant DOSE EQUIVALENT I-131 Specific Activity Limit versus Percent of RATED THERMAL POWER" and insert a limit of less than or equal to the site specific DEI spiking limit. Radiological dose consequence analyses for SGTR and MSLB accidents, which take into account the pre-accident iodine spike, do not consider the elevated RCS iodine specific activities permitted by Figure 3.4.16-1 for operation below 80% RTP. Instead, the pre-accident iodine spike analyses assume a DEI concentration 60 times higher than the corresponding long term equilibrium value, which corresponds to the specific activity limit associated with 100% RTP operation. TS 3.4.16 Required Action A.1 shall be based on the short term site specific DEI spiking limit to be consistent with the assumptions contained in the radiological consequence analyses.

#### 3.1.4 TS 3.4.16 Applicability Revision

TS 3.4.16 Applicability is modified to include MODE 3 and MODE 4. It is necessary for the LCO to apply during MODES 1 through 4 to limit the potential radiological consequences of a SGTR or MSLB that may occur during these MODES. In MODES 5 and 6, the steam generators are not used for decay heat removal, the RCS and steam generators are depressurized, and primary to secondary leakage is minimal. Therefore, the monitoring of RCS specific activity during MODES 5 and 6 is not required. The change to modify the TS 3.4.16 Applicability to include all of MODE 3 and MODE 4 is necessary to limit the potential radiological consequences of an SGTR or MSLB that may occur during these MODES and is therefore acceptable from a radiological dose perspective.

#### 3.1.5 TS 3.4.16 Condition A Revision

TS 3.4.16 Condition A is revised by replacing the DEI site specific limit "> 1.0  $\mu\text{Ci/gm}$ " with the words "not within limit" to be consistent with the revised TS 3.4.16 LCO format. The site specific DEI limit of  $\leq [1.0] \mu\text{Ci/gm}$  and Required Action A.1 is contained in SR 3.4.16.2. This change will maintain the consistency of the proposed TS and is acceptable from a radiological dose perspective.

#### 3.1.6 TS 3.4.16 Condition B Revision to include Action for DEX Limit

TS 3.4.16 Condition C is replaced with a new Condition B for DEX not within limits. This change is made to be consistent with the change to the TS 3.4.16 LCO which requires the DEX specific activity to be within limits as discussed above. The DEX limit is site specific and the numerical value in units of  $\mu\text{Ci/gm}$  is contained in revised SR 3.4.16.1. The site specific limit of DEX  $\mu\text{Ci/gm}$  is established based on the maximum accident analysis RCS activity corresponding to 1% fuel clad defects with sufficient margin to accommodate the exclusion of those isotopes

based on low concentration, short half life, or small dose conversion factors. The primary purpose of the TS 3.4.16 LCO on RCS specific activity and its associated Conditions is to support the dose analyses for DBAs. The whole body dose is primarily dependent on the noble gas activity, not the non-gaseous activity currently captured in the E-Bar definition.

The Completion Time for revised TS 3.4.16 Required Action B.1 will require restoration of DEX to within limit in 48 hours. This is consistent with the Completion Time for current Required Action A.2 for DEI. The radiological consequences for the SGTR and the MSLB accidents demonstrate that the calculated thyroid doses are generally a greater percentage of the applicable acceptance criteria than the calculated whole body doses (operation with iodine specific activity levels greater than the LCO limit is permissible, if the activity levels do not exceed the limits shown in Figure 3.4.16-1, in the applicable specification, for more than 48 hours). Therefore the Completion Time for noble gas activity being out of specification in the revised Required Action B.1 should be at least as great as the Completion Time for iodine specific activity being out of specification in current Required Action A.2. Therefore the Completion Time of 48 hours for revised Required Action B.1 is acceptable from a radiological dose perspective.

#### 3.1.7 TS 3.4.16 Condition C Revision

TS 3.4.16 Condition C is revised to include Condition B (DEX not within limit) if the Required Action and associated Completion Time of Condition B is not met. This is consistent with the changes made to Condition B which now provides the same completion time for both components of RCS specific activity as discussed in the revision to Condition B. The revision to Condition C also replaces the limit on DEI from the deleted Figure 3.4.16-1 with a site specific value of  $> [60] \mu\text{Ci/gm}$ . This change makes Condition C consistent with the changes made to TS 3.4.16 Required Action A.1.

The change to TS 3.4.16 Required Action C.1 requires the plant to be in MODE 3 within 6 hours and adds a new Required Action C.2 which requires the plant to be in MODE 5 within 36 hours. These changes are consistent with the changes made to the TS 3.4.16 Applicability. The revised LCO is applicable throughout all of MODES 1 through 4 to limit the potential radiological consequences of an SGTR or MSLB that may occur during these MODES. In MODES 5 and 6, the steam generators are not used for decay heat removal, the RCS and steam generators are depressurized, and primary to secondary leakage is minimal. Therefore, the monitoring of RCS specific activity during MODES 5 and 6 is not required.

A new TS 3.4.16 Required Action C.2 Completion Time of 36 hours is added for the plant to reach MODE 5. This Completion Time is reasonable, based on operating experience, to reach MODE 5 from full power conditions in an orderly manner and without challenging plant systems and the value of 36 hours is consistent with other TS which have a Completion Time to reach MODE 5.

#### 3.1.8 SR 3.4.16.1 Revision to include Surveillance for DEX

The change replaces the current SR 3.4.16.1 surveillance for RCS gross specific activity with a surveillance to verify that the site specific reactor coolant DEX specific activity is  $\leq [280] \mu\text{Ci/gm}$ . This change provides a surveillance for the new LCO limit added to TS 3.4.16 for DEX. The revised SR 3.4.16.1 surveillance requires performing a gamma isotopic analysis as a measure of the noble gas specific activity of the reactor coolant at least once every 7 days which is the same frequency required under the current SR 3.4.16.1 surveillance for RCS gross non-iodine specific activity. The surveillance provides an indication of any increase in the noble gas specific activity. The results of the surveillance on DEX allow proper remedial action to be taken before reaching the LCO limit under normal operating conditions.

SR 3.4.16.1 is modified by inclusion of a NOTE which permits the use of the provisions of LCO 3.0.4.c. This allowance permits entry into the applicable MODE(S) while relying on the ACTIONS. This allowance is acceptable due to the significant conservatism incorporated into the specific activity limit, the low probability of an event which is limiting due to exceeding this limit, and the ability to restore transient specific activity excursions while the plant remains at, or proceeds to power operation. This allows entry into MODE 4, MODE 3, and MODE 2 prior to performing the surveillance. This allows the surveillance to be performed in any of those MODES, prior to entering MODE 1, similar to the current surveillance SR 3.4.16.2 for DEI.

#### 3.1.9 SR 3.4.16.3 Deletion

The current SR 3.4.16.3 which required the determination of E-Bar is deleted. TS 3.4.16 LCO on RCS specific activity supports the dose analyses for DBAs, in which the whole body dose is primarily dependent on the noble gas concentration, not the non-gaseous activity currently captured in the E-Bar definition. With the elimination of the limit for RCS gross specific activity and the addition of the new LCO limit for noble gas specific activity, this SR to determine E-Bar is no longer required.

#### 3.2 Precedent

The Technical Specifications developed for the Westinghouse AP600 and AP1000 advanced reactor designs incorporate an LCO for RCS DEX activity in place of the LCO on non-iodine gross specific activity based on E-Bar. This approach was approved by the NRC for the AP600 in NUREG-1512, "Final Safety Evaluation Report Related to the Certification of the AP600 Standard Design, Docket No. 52-003," dated August 1998 and for the AP1000 in the NRC letter to Westinghouse Electric Company dated September 13, 2004. In addition the curve describing the maximum allowable iodine concentration during the 48-hour period of elevated activity as a function of power level, was not included in the TS approved for the AP600 and AP1000 advanced reactor designs.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the [ ] State official was notified of the proposed issuance of the amendment. The State official had [(1) no comments or (2) the following comments—with subsequent disposition by the staff].

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment[s] change[s] a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding published [DATE] ([ ] FR [ ]). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

#### Proposed No Significant Hazards Consideration Determination

*Description of Amendment Request:* [LICENSEE] requests adoption of an approved change to the standard technical specifications (STS) for Pressurized Water Reactor (PWR) Plants [NUREG-1430, NUREG-1431, or NUREG-1432] and the [LICENSEE] technical specifications (TS), to replace the current limits on primary coolant gross specific activity with limits on primary coolant noble gas activity. The noble gas activity would be based on DOSE EQUIVALENT XE-133 and would take into account only the noble gas activity in the primary coolant. The changes are consistent with NRC approved Industry/Technical Specification Task Force (TSTF) Standard Technical Specification Change Traveler, TSTF-490.

*Basis for proposed no-significant-hazards-consideration determination:* As required by 10 CFR 50.91(a), an analysis of the issue of no-significant-hazards-consideration is presented below:

*Criterion 1—The Proposed Change Does Not Involve a Significant Increase in the Probability or Consequences of an Accident Previously Evaluated*

Reactor coolant specific activity is not an initiator for any accident previously evaluated. The Completion Time when primary coolant gross activity is not within limit is not an initiator for any accident previously evaluated. The current variable limit on primary coolant iodine concentration is not an initiator to any accident previously evaluated. As a result, the proposed change does not significantly increase the probability of an accident. The proposed change will limit primary coolant noble gases to concentrations consistent with the accident analyses. The proposed change to the Completion Time has no impact on the consequences of any design basis accident since the consequences of an accident during the extended Completion Time are the same as the consequences of an accident during the Completion Time. As a result, the consequences of any accident previously evaluated are not significantly increased.

*Criterion 2—The Proposed Change Does Not Create the Possibility of a New or Different Kind of Accident from any Previously Evaluated*

The proposed change in specific activity limits does not alter any physical part of the plant nor does it affect any plant operating parameter. The change does not create the potential for a new or different kind of accident from any previously calculated.

*Criterion 3—The Proposed Change Does Not Involve a Significant Reduction in the Margin of Safety*

The proposed change revises the limits on noble gas radioactivity in the primary coolant. The proposed change is consistent with the assumptions in the safety analyses and will ensure the monitored values protect the initial assumptions in the safety analyses.

Based upon the reasoning presented above and the previous discussion of the amendment request, the requested change does not involve a significant hazards consideration.

Dated at Rockville, Maryland, this \_\_\_ day of \_\_\_\_\_, XXXX.

For The Nuclear Regulatory Commission  
Project Manager, Plant Licensing Branch  
[ ], Division of Operating Reactor Licensing,  
Office of Nuclear Reactor Regulation

Enclosure 2—[Proposed Technical Specification Changes and Technical Specification Bases Changes]

Enclosure 3—[Final Technical Specification and Bases Pages]  
[FR Doc. 06-9330 Filed 11-17-06; 8:45 am]

**BILLING CODE 7590-01-P**

#### OVERSEAS PRIVATE INVESTMENT CORPORATION

##### Agency Information Collection Activities: Proposed Collection; Comment Request

**AGENCY:** Overseas Private Investment Corporation (OPIC).

**ACTION:** Request for comments.

**SUMMARY:** In compliance with the Paperwork Reduction Act (44 U.S.C. 3501 *et seq.*), this notice announces that OPIC is planning to submit the following proposed Information Collection Request (ICR) to the Office of Management and Budget (OMB): OPIC 2006 Client Satisfaction Survey. Before submitting the ICR to OMB for review and approval, OPIC is soliciting public comment on the client survey. Comments are being solicited on the need for the information, its practical utility, the accuracy of the Agency's burden estimate, and on ways to minimize the reporting burden. The proposed ICR is summarized below.

**DATES:** All comments must be received by OPIC within 60 calendar days from the publication date of this Notice.

**ADDRESSES:** Information regarding this information collection request and/or a copy of the survey questions can be obtained from the Agency Submitting Officer below. Comments on the survey should be submitted to the Agency Submitting Officer.

**FOR FURTHER INFORMATION CONTACT:**  
OPIC Agency Submitting Officer: Essie Bryant, Records Manager, Overseas Private Investment Corporation, 1100 New York Avenue, NW., Washington, DC 20527, telephone (202) 336-8563.

##### SUPPLEMENTARY INFORMATION:

*Type of Request:* New information collection.

*Title:* 2006 OPIC Client Satisfaction Survey.

*OMB Approval Number:* None.

*Frequency of Response:* Once per client.

*Type of Respondents:* Individual business officer representatives of U.S. companies sponsoring projects overseas.

*Respondent's Obligation:* Voluntary.

*Affected Public:* U.S. companies or citizens sponsoring projects overseas.

*Estimated Number of Respondents:* 100.

*Estimated time Per Response:* 30 minutes.

*Estimated Total Annual Burden:* \$0.

*Authority for Information Collection:* Sections 231 and 234 of the Foreign Assistance Act of 1961, as amended.

*Abstract (Needs and Uses of Information Collected):* OPIC is