renewal of Facility Operating License No. R–110 and has concluded that this action will not have a significant effect on the quality of the human environment.

For further details with respect to this action, see: (1) The application for amendment dated November 21, 1995, as supplemented on January 31, 2003 and July 10, 2003, (2) Amendment No. 6 to Facility Operating License No. R–110; (3) the related Safety Evaluation Report and (4) the Environmental Assessment dated March 30, 2004. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. The NRC maintains an Agencywide Documents Access and Management System (ADAMS), which provides text and image files of NRC's public documents. Documents related to this license renewal dated on or after November 24, 1999, may be accessed through the NRC's Public Electronic Reading Room on the Internet at *http://www.nrc.gov*. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1–800–397–4209, 301–415–4737 or by e-mail to *pdr@nrc.gov*.

Dated at Rockville, Maryland, this 14th day of August 2006.

For the U.S. Nuclear Regulatory Commission.

Brian E. Thomas,

Chief, Research and Test Reactors Branch, Division of Policy and Rulemaking, Office of Nuclear Reactor Regulation.

[FR Doc. E6–15310 Filed 9–14–06; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Draft Regulatory Guide and Associated Standard Review Plan: Issuance, Availability

The U.S. Nuclear Regulatory Commission (NRC) has issued for public comment a draft proposed revision of an existing guide in the agency's Regulatory Guide Series. This series has been developed to describe and make available to the public such information as methods that are acceptable to the NRC staff for implementing specific parts of the agency's regulations, techniques that the staff uses in evaluating specific problems or postulated accidents, and data that the staff needs in its review of applications for permits and licenses.

This draft Revision 1 of Regulatory Guide 1.200, ''An Approach for Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities,'' is temporarily identified as Draft Regulatory Guide DG–1161, which should be mentioned in all related correspondence. Like its predecessors, this proposed revision describes one acceptable approach for determining whether the quality of a probabilistic risk assessment (PRA), in total or the parts that are used to support an application, is sufficient to provide confidence in the results, such that the PRA can be used in regulatory decision-making for light-water reactors. Specifically, Draft Regulatory Guide DG–1161 provides guidance in four areas:

(1) A minimal set of functional requirements of a technically acceptable PRA.

(2) The NRC's position on PRA consensus standards and industry PRA program documents.

(3) Demonstration that the PRA (in total or specific parts) used in regulatory applications is of sufficient technical adequacy.

(4) Documentation to support a regulatory submittal.

This guidance is intended to be consistent with the NRC's PRA Policy Statement, entitled ''Use of Probabilistic Risk Assessment Methods in Nuclear Activities: Final Policy Statement,'' which the NRC published in the **Federal Register** on August 16, 1995 (60 FR 42622) to encourage use of PRA in all regulatory matters. That Policy Statement states that '' * * * the use of PRA technology should be increased to the extent supported by the state-of-the-art in PRA methods and data and in a manner that complements the NRC's deterministic approach.'' Since that time, many uses have been implemented or undertaken, including modification of the NRC's reactor safety inspection program and initiation of work to modify reactor safety regulations. Consequently, confidence in the information derived from a PRA is an important issue, in that the accuracy of the technical content must be sufficient to justify the specific results and insights that are used to support the decision under consideration.

Draft Regulatory Guide DG–1161 is also intended to be consistent with the more detailed, guidance in Regulatory Guide 1.174, ''An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis,'' which the NRC issued in November 2002. In addition, Draft Regulatory Guide DG–1161 is intended to reflect and endorse (with certain objections) the following guidance provided by the American Society of Mechanical Engineers (ASME) and the Nuclear Energy Institute (NEI):

- ASME RA–S–2002, ''Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications,'' dated April 5, 2002.
- ASME RA–Sa–2003, ''Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications,'' Addendum A to ASME RA–S–2002, dated December 5, 2003.
- ASME RA–Sb–2005, ''Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications,'' Addendum B to ASME RA–S–2002, dated December 30, 2005.
- NEI–00–02, ''Probabilistic Risk Assessment Peer Review Process Guidance,'' Revision A3, dated March 20, 2000, with its supplemental guidance on industry self-assessment, dated August 16, 2002, and Revision 1, dated May 19, 2006.
- NEI–05–04, ''Process for Performing Follow-on PRA Peer Reviews Using the ASME PRA Standard,'' dated January 2005.

When used in support of an application, this regulatory guide will obviate the need for an in-depth review of the base PRA by NRC reviewers, allowing them to focus their review on key assumptions and areas identified by peer reviewers as being of concern and relevant to the application. Consequently, this guide will provide for a more focused and consistent review process. In this regulatory guide, as in RG 1.174, the quality of a PRA analysis used to support an application is measured in terms of its appropriateness with respect to scope, level of detail, and technical acceptability.

This regulatory guide was issued for trial use in February of 2004, and five trial applications were conducted. This revision incorporates lessons learned from those pilot applications. In addition, the appendices to this regulatory guide have been revised to address the changes made in the professional society PRA standards and industry PRA guidance documents.

To accompany Draft Regulatory Guide DG–1161, the NRC is issuing proposed Revision 2 of Section 19.1, ''Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities,'' of NUREG–0800, ''Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants'' (SRP). This SRP complements Draft

Regulatory Guide DG–1161, in that the NRC staff will use its guidance to ensure more focused and consistent review of PRAs as a basis for regulatory decision-making for light-water reactors.

The NRC intends to update Regulatory Guide 1.200 and its associated SRP Section 19.1, and to develop an additional appendix or revise an existing appendix (as required), to set forth the staff's position when a new or revised PRA standard or industry program is published.

The NRC staff is soliciting comments on Draft Regulatory Guide DG–1161, as well as draft Revision 2 of SRP Section 19.1. Please mention the relevant document identifiers (DG–1161 and/or SRP 19.1) in the subject line of your comments; comments may be accompanied by relevant information or supporting data. Comments submitted in writing or in electronic form will be made available to the public in their entirety through the NRC's Agencywide Documents Access and Management System (ADAMS). Personal information will not be removed from your comments. You may submit comments by any of the following methods.

Mail comments to: Rules and Directives Branch, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001.

E-mail comments to: NRCREP@nrc.gov. You may also submit comments via the NRC's rulemaking Web site at *http://ruleforum.llnl.gov*. Address questions about our rulemaking Web site to Carol A. Gallagher (301) 415–5905; e-mail *CAG@nrc.gov*.

Hand-deliver comments to: Rules and Directives Branch, Office of Administration, U.S. Nuclear Regulatory Commission, 11555 Rockville Pike, Rockville, Maryland 20852, between 7:30 a.m. and 4:15 p.m. on Federal workdays.

Fax comments to: Rules and Directives Branch, Office of Administration, U.S. Nuclear Regulatory Commission at (301) 415–5144.

Requests for technical information about Draft Regulatory Guide DG–1161 and/or draft Revision 2 of SRP Section 19.1 may be directed to Ms. Mary T. Drouin, at (301) 415–6675 or *MXD@nrc.gov*.

Comments would be most helpful if received by October 14, 2006. Comments received after that date will be considered if it is practical to do so, but the NRC is able to ensure consideration only for comments received on or before this date. Although a time limit is given, comments and suggestions in connection with items for inclusion in guides currently being developed or improvements in all published guides are encouraged at any time.

Electronic copies of Draft Regulatory Guide DG–1161 are available through the NRC's public Web site under Draft Regulatory Guides in the Regulatory Guides document collection of the NRC's Electronic Reading Room at *http://www.nrc.gov/reading-rm/doc-collections/*. Similarly, electronic copies of draft Revision 2 of SRP Section 19.1 are available at *http://www.nrc.gov/reading-rm/doc-collections/nuregs/docs4comment.html*. Electronic copies of the two documents are also available in ADAMS at *http://www.nrc.gov/reading-rm/adams.html*, under Accession #ML062480134 and #ML062510220, respectively.

In addition, Draft Regulatory Guide DG–1161, draft Revision 2 of SRP Section 19.1, and other related publicly available documents, including public comments received, can be viewed electronically on computers in the NRC's Public Document Room (PDR), which is located at 11555 Rockville Pike, Rockville, Maryland. The PDR reproduction contractor will make copies of documents for a fee. The PDR's mailing address is USNRC PDR, Washington, DC 20555–0001. The PDR can also be reached by telephone at (301) 415–4737 or (800) 397–4205, by fax at (301) 415–3548, and by e-mail to *PDR@nrc.gov*.

Please note that the NRC does not intend to distribute printed copies of either Draft Regulatory Guide DG–1161 or draft Revision 2 of SRP Section 19.1, unless specifically requested on an individual basis with adequate justification. Such requests for single copies of draft or final guides (which may be reproduced) or for placement on an automatic distribution list for single copies of future draft guides in specific divisions should be made in writing to the U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, *Attention:* Reproduction and Distribution Services Section; by e-mail to *DISTRIBUTION@nrc.gov*; or by fax to (301) 415–2289. Telephone requests cannot be accommodated.

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(5 U.S.C. 552(a))

Dated at Rockville, Maryland, this 8th day of September 2006.

For the U.S. Nuclear Regulatory Commission.

Farouk Eltawila,

Director, Division of Risk Assessment and Special Projects, Office of Nuclear Regulatory Research.

[FR Doc. E6–15311 Filed 9–14–06; 8:45 am]

BILLING CODE 7590–01–P

PENSION BENEFIT GUARANTY CORPORATION

Required Interest Rate Assumption for Determining Variable-Rate Premium for Single-Employer Plans; Interest Assumptions for Multiemployer Plan Valuations Following Mass Withdrawal

AGENCY: Pension Benefit Guaranty Corporation.

ACTION: Notice of interest rates and assumptions.

SUMMARY: This notice informs the public of the interest rates and assumptions to be used under certain Pension Benefit Guaranty Corporation regulations. These rates and assumptions are published elsewhere (or can be derived from rates published elsewhere), but are collected and published in this notice for the convenience of the public. Interest rates are also published on the PBGC's Web site (*http://www.pbgc.gov*).

DATES: The required interest rate for determining the variable-rate premium under part 4006 applies to premium payment years beginning in September 2006. The interest assumptions for performing multiemployer plan valuations following mass withdrawal under part 4281 apply to valuation dates occurring in October 2006.

FOR FURTHER INFORMATION CONTACT: Catherine B. Klion, Manager, Regulatory and Policy Division, Legislative and Regulatory Department, Pension Benefit Guaranty Corporation, 1200 K Street, NW., Washington, DC 20005, 202–326–4024. (TTY/TDD users may call the Federal relay service toll-free at 1–800–877–8339 and ask to be connected to 202–326–4024.)

SUPPLEMENTARY INFORMATION:

Variable-Rate Premiums

Section 4006(a)(3)(E)(iii)(II) of the Employee Retirement Income Security Act of 1974 (ERISA) and § 4006.4(b)(1) of the PBGC's regulation on Premium Rates (29 CFR part 4006) prescribe use of an assumed interest rate (the ''required interest rate'') in determining a single-employer plan's variable-rate premium. Pursuant to the Pension Protection Act of 2006, for premium payment years beginning in 2006 or 2007, the required interest rate is the