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Documents related to operations conducted under these licenses not specifically referenced in this Notice may not be electronically available and/or may not be publicly available. Persons who have an interest in reviewing these documents should submit a request to NRC under the Freedom of Information Act (FOIA). Instructions for submitting a FOIA request can be found on the NRC's Web site at <http://www.nrc.gov/reading-rm/foia/foia-privacy.html>.

Dated at King of Prussia, Pennsylvania this 23rd day of March, 2006.

For the Nuclear Regulatory Commission.

James P. Dwyer,

Chief, Commercial and R&D Branch, Division of Nuclear Materials Safety, Region 1.

[FR Doc. E6-5261 Filed 4-10-06; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 72-28]

PPL Susquehanna, LLC; Independent Spent Fuel Storage Installation; Environmental Assessment and Finding of No Significant Impact

AGENCY: Nuclear Regulatory Commission.

ACTION: Issuance of an environmental assessment and Finding of No Significant Impact.

FOR FURTHER INFORMATION CONTACT: Joseph M. Sebrosky, Senior Project Manager, Spent Fuel Project Office,

Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Telephone: (301) 415-1132; Fax number: (301) 415-8555; e-mail: jms3@nrc.gov.

SUPPLEMENTARY INFORMATION: The U.S. Nuclear Regulatory Commission (NRC or Commission) is considering issuance of an exemption to PPL Susquehanna, LLC (PPL) pursuant to 10 CFR 72.7, from specific provisions of 10 CFR 72.212(a)(2), 72.212(b)(2)(i)(A), 72.212(b)(7), and 72.214. The licensee wants to use the Transnuclear, Inc. (TN) NUHOMS® Storage System, Certificate of Compliance No. 1004 (CoC or Certificate) Amendment No. 8 (61BT dry shielded canister), to store spent nuclear fuel under a general license in an Independent Spent Fuel Storage Installation (ISFSI) associated with the operation of the Susquehanna Steam Electric Station (SSES), Units 1 and 2, located in Luzerne County, Pennsylvania. PPL is requesting an exemption from CoC No. 1004 to allow loading of Framatome ANP 9x9-2 fuel assemblies in the NUHOMS®-61BT dry shielded canister (DSC).

Environmental Assessment (EA)

Identification of Proposed Action: The proposed action would exempt PPL from the requirements of 10 CFR 72.212(a)(2), 72.212(b)(2)(i)(A), 72.212(b)(7), and 72.214 and enable PPL to use the TN NUHOMS®-61BT DSC with modifications at SSES. These regulations specifically require storage in casks approved under the provisions of 10 CFR part 72 and compliance with the conditions set forth in the CoC for each dry spent fuel storage cask used by an ISFSI general licensee. The TN NUHOMS® CoC provides requirements, conditions, and operating limits in Attachment A, Technical Specifications. The proposed action would exempt PPL from the requirements of 10 CFR 72.212(a)(2), 10 CFR 72.212(b)(7) and 10 CFR 72.214 from a condition in Amendment 8 to CoC No. 1004 so that Framatome ANP 9x9-2 fuel assemblies can be loaded in a NUHOMS®-61BT DSC. Specifically, the exemption would be from CoC No. 1004 Attachment A, Technical Specification, Table 1-1d, "BWR Fuel Assembly Design Characteristics for the NUHOMS®-61BT DSC," which allows for the storage of General Electric (or equivalent) 9x9-2 fuel assemblies that contain 66 full and 8 partial fuel rods. The exemption would allow PPL to store Framatome ANP 9x9-2 fuel assemblies that contain 79 full fuel rods and no partial fuel rods in the NUHOMS®-61BT DSC.

PPL committed in its January 31, 2006, submittal to a maximum decay heat load per fuel assembly of 210 watts. This is less than the CoC No. 1004 Attachment A, Technical Specification, Table 1-1c maximum decay heat limit of 300 watts per assembly. In addition, in its March 6, 2006, supplement PPL provided the parameters found in Table 1 below associated with the Framatome ANP 9x9-2 fuel assembly.

TABLE 1.—PARAMETERS FOR FRAMATOME ANP 9X9-2 FUEL ASSEMBLY

Manufacturer	Framatome ANP.
Version	FANP9.
Number of Fuel Rods per Assembly.	79 full.
Fuel Pellet Outside Diameter (inches).	0.3565.
Clad Outside Diameter (inches).	0.424.
Water Rod Inside Diameter (inches).	0.364.
Array	9x9.
Active Fuel Length (inches).	150.
Pitch (inches)	0.572.
Clad Thickness (inches).	0.030.
Water Rod Outside Diameter (inches).	0.425.

The NRC has determined that the exemption, if granted, will contain the following 3 conditions:

(1) PPL will be limited to loading a total of five 61BT DSCs under this exemption if granted.

(2) PPL shall limit the decay heat level per fuel assembly to 210 watts to ensure cask loadings are bounded by the analyses supporting TN CoC No. 1004, Amendment No. 8., and (3) the exemption will pertain only to Framatome ANP 9x9-2 fuel assemblies that meet the nominal un-irradiated design parameters contained in Table 1 above.

The proposed action is in accordance with the licensee's request for exemption dated January 31, 2006, as supplemented March 6, 2006.

Need for the Proposed Action: The proposed action is needed because SSES will lose full core offload capability in December 2006 following the receipt and staging of new fuel for the scheduled 2007 Unit 2 refueling outage. PPL has determined that it is necessary to start the dry fuel storage (DFS) campaign in May 2006 to ensure full core offload capability. PPL had originally scheduled a DFS campaign to begin in October of 2006. However, because of recent SSES Unit 1 fuel channel performance problems, 54 fuel channels were replaced and stored in

the spent fuel pool. As a result of this fuel channel problem, a possible Unit 2 mid-cycle maintenance outage may be necessary to inspect and replace, if necessary, any affected fuel channels. This mid-cycle outage is tentatively scheduled for the Fall of 2006 and access to the spent fuel pool is needed to store fuel channels that are replaced. This activity would conflict with loading dry fuel storage casks. There is also a conflict with performing the DFS campaign in the Summer of 2006. Specifically, PPL has contracted to perform a spent fuel pool cleanout beginning in June 2006 so adequate pool space is restored to support the Unit 2 2007 refueling outage. The DFS campaign and the spent fuel pool cleanout campaign cannot occur simultaneously. Rescheduling the spent fuel cleanout campaign for later in the year is difficult. In summary, space available in the spent fuel pool has become limited much sooner than anticipated, and PPL is requesting the exemption to support a DFS campaign in May 2006. A DFS campaign in May 2006 will also allow PPL flexibility for fuel storage options related to managing decay heat loads within the spent fuel pool.

The proposed action is necessary because the NRC has not yet received an amendment to CoC No. 1004 to allow loading of a Framatome ANP 9x9-2 in a NUHOMS® 61BT DSC. The staff would have to review such an amendment request and only after making the appropriate findings would the staff initiate 10 CFR 72.214 rulemaking to implement the change. This process typically takes at least 10 months from the receipt of the amendment request for simple license amendments. Complex license amendments can take over 30 months. Therefore, an amendment to allow loading of Framatome ANP 9x9-2 fuel assemblies in the NUHOMS® -61BT DSC can not be completed in time to support PPL's stated needs.

Environmental Impacts of the Proposed Action: The NRC has completed its evaluation of the proposed action and concludes that there will be no significant environmental impact if the exemption is granted. The staff has determined that the proposed action would not endanger life or property. The potential environmental impact of using the NUHOMS® system was initially presented in the Environmental Assessment (EA) for the Final Rule to add the TN Standardized NUHOMS® Horizontal Modular Storage System for Irradiated Nuclear Fuel to the list of approved spent fuel storage casks in 10

CFR 72.214 (59 FR 65898, dated December 22, 1994). The potential environmental impact of using the NUHOMS® -61BT DSC was initially presented in the Environmental Assessment (EA) for the Final Rule to add the 61BT DSC to the Standardized NUHOMS® system, Amendment No. 3 (66 FR 34523, dated June 29, 2001).

The staff performed a safety evaluation of the proposed exemption. The staff has determined that the Framatome ANP 9x9-2 assemblies, which PPL plans to load into the 61BT DSC are bounded by the design basis fuel assemblies for the 61BT DSC previously evaluated by the staff. The staff's thermal safety evaluation review notes that PPL committed to only loading Framatome ANP 9x9-2 fuel assemblies with a maximum decay heat load per assembly of 210 watts. This is less than the CoC No. 1004 Attachment A, Technical Specification, Table 1-1c maximum decay heat limit of 300 watts per assembly and is therefore bounding. In the criticality area, the staff evaluated the criticality code and the selected cross sectional data that were used by PPL and determined that they were sufficiently documented and validated, and that they are appropriate for the Framatome 9x9-2 fuel assembly. The staff also performed independent confirmatory criticality calculations for normal conditions of storage and transfer based on the parameters for the Framatome ANP 9x9-2 fuel assembly identified in Table 1 above. Based on its review of the representations and information supplied by the applicant, and the confirmatory analyses performed by staff, the staff concludes that the nuclear criticality safety design has been adequately described and evaluated by the applicant, and finds reasonable assurance that the Framatome ANP 9x9-2 fuel meets the criticality safety requirements of 10 CFR part 72.

The loading of Framatome ANP 9x9-2 fuel assemblies in the NUHOMS® 61BT DSC does not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released offsite, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

The exemption only affects the requirements associated with the fuel assemblies that can be loaded in the casks and does not affect non-radiological plant effluents or any other aspects of the environment. Therefore, there are no significant non-radiological

impacts associated with the proposed action.

Accordingly, the Commission concludes that there are no significant environmental impacts associated with the proposed action.

Alternative to the Proposed Action: Because there is no significant environmental impact associated with the proposed action, alternatives with equal or greater environmental impact were not evaluated. As an alternative to the proposed action, the staff considered denial of the proposed action. Denial of the exemption would result in no change in the current environmental impact.

Agencies and Persons Consulted: This exemption request was discussed with Mr. Brad Fuller of the Pennsylvania Department of Environmental Protection Bureau of Radiation Protection on March 6, 2006. He stated that the State had no comments on the technical aspects of the exemption. The NRC staff has determined that a consultation under Section 7 of the Endangered Species Act is not required because the proposed action will not affect listed species or critical habitat. The NRC staff has also determined that the proposed action is not a type of activity having the potential to cause effects on historic properties. Therefore, no further consultation is required under Section 106 of the National Historic Preservation Act.

Conclusion: The staff has reviewed the exemption request submitted by PPL. Allowing loading of Framatome 9x9-2 fuel assemblies in the NUHOMS® 61BT DSC would have no significant impact on the environment.

Finding of No Significant Impact

The environmental impacts of the proposed action have been reviewed in accordance with the requirements set forth in 10 CFR part 51. Based upon the foregoing Environmental Assessment, the Commission finds that the proposed action of granting the exemption from specific provisions of 10 CFR 72.212(a)(2), 72.212(b)(2)(i)(A), 72.212(b)(7), and 10 CFR 72.214, to allow PPL to load of Framatome ANP 9x9-2 fuel assemblies in the NUHOMS® 61BT DSC, subject to conditions, will not significantly impact the quality of the human environment. Accordingly, the Commission has determined that an environmental impact statement for the proposed exemption is not warranted.

In accordance with 10 CFR 2.390 of NRC's "Rules of Practice," final NRC records and documents regarding this proposed action are publically available in the records component of NRC's Agencywide Documents Access and

Management System (ADAMS). The request for exemption dated January 31, 2006, and March 6, 2006, was docketed under 10 CFR part 72, Docket No. 72-28. These documents may be inspected at NRC's Public Electronic Reading Room at <http://www.nrc.gov/reading-rm/adams.html>. These documents may also be viewed electronically on the public computers located at the NRC's Public Document Room (PDR), O1F21, One White Flint North, 11555 Rockville Pike, Rockville, MD 20852. The PDR reproduction contractor will copy documents for a fee. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS, should contact the NRC PDR Reference staff by telephone at 1-800-397-4209 or (301) 415-4737, or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 31st day of March, 2006.

For the Nuclear Regulatory Commission.

Joseph M. Sebrosky,

Senior Project Manager, Spent Fuel Project Office, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 06-3416 Filed 4-10-06; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 52-009]

System Energy Resources, Inc. Notice of Availability of the Final Environmental Impact Statement for an Early Site Permit (ESP) at the Grand Gulf ESP Site

Notice is hereby given that the U.S. Nuclear Regulatory Commission (NRC or the Commission) has published NUREG-1817, "Environmental Impact Statement for an Early Site Permit (ESP) at the Grand Gulf ESP Site—Final Report." The site is located near the Town of Port Gibson in Claiborne County, Mississippi. The application for the ESP was submitted by letter dated October 16, 2003, pursuant to Title 10 of the *Code of Federal Regulations* Part 52 (10 CFR Part 52). A notice of receipt and availability of the application, which included the environmental report (ER), was published in the **Federal Register** on November 14, 2003 (68 FR 64665). A notice of acceptance for docketing of the application for the ESP was published in the **Federal Register** on December 1, 2003 (68 FR 67219). A notice of intent to prepare an environmental impact statement (EIS) and to conduct the scoping process was published in the **Federal Register** on December 31, 2003 (68 FR 75656). A

notice of availability of the draft EIS was published in the **Federal Register** on April 28, 2005 (70 FR 22155).

The purpose of this notice is to inform the public that NUREG-1817, "Environmental Impact Statement for an Early Site Permit (ESP) at the Grand Gulf ESP Site—Final Report," is available for public inspection in the NRC Public Document Room (PDR) located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland, 20852, or from the Publicly Available Records (PARS) component of NRC's Agencywide Documents Access and Management System (ADAMS), and will also be placed directly on the NRC Web site at <http://www.nrc.gov>. ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room). Persons who do not have access to ADAMS, or who encounter problems in accessing the documents located in ADAMS, should contact the PDR reference staff at 1-800-397-4209, 301-415-4737, or by e-mail to pdr@nrc.gov. In addition, the Harriette Person Memorial Library, located at 606 Main Street, Port Gibson, Mississippi, has agreed to make the final EIS available for public inspection.

FOR FURTHER INFORMATION CONTACT:

James H. Wilson, Environmental Branch A, Division of License Renewal, U.S. Nuclear Regulatory Commission, Washington, DC, 20555-0001. Mr. Wilson may be contacted by telephone at 301-415-1108 or by e-mail at jhw1@nrc.gov.

Dated at Rockville, Maryland, this 3rd day of April 2006.

For the Nuclear Regulatory Commission

Frank P. Gillespie,

Director, Division of License Renewal, Office of Nuclear Reactor Regulation.

[FR Doc. E6-5256 Filed 4-10-06; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on the Medical Uses of Isotopes: Meeting Notice

AGENCY: Nuclear Regulatory Commission.

ACTION: Updated notice of meeting.

SUMMARY: The U.S. Nuclear Regulatory Commission will convene a meeting of the Advisory Committee on the Medical Uses of Isotopes (ACMUI) on April 25 and 26, 2006. A sample of agenda items to be discussed during the public sessions includes: (1) Updates on Proposed Regulations to Include Discrete Radium Sources and

Accelerator-Produced Radioactive Materials in 10 CFR Part 35; (2) RIS on Visitor Dose Limits; (3) Part 35, Training and Experience; (4) Supply of High Enriched Uranium for Molybdenum-99 Generation; (5) Training and Experience for Use of Microspheres for Therapy; (6) ACMUI Review of Medical Events Involving I-131. To review the agenda see: <http://www.nrc.gov/reading-rm/doc-collections/acmui/agenda/> or contact, via e-mail: mss@nrc.gov.

Purpose: Discuss issues related to 10 CFR 35, Medical Use of Byproduct Material.

Date and Time for Closed Session Meeting: April 25, 2006, from 8 a.m. to 10:15 a.m. This session will be closed so that NRC staff can brief the ACMUI on information relating solely to internal personnel rules and can discuss protected information of an investigatory nature. Time may be added to the closed session or an additional closed session may be added as needed.

Dates and Times for Public Meetings: April 25, 2006, from 10:30 a.m. to 5 p.m.; and April 26, 2006, from 8 a.m. to 11:30 a.m.

ADDRESSES: *Address for Public Meetings:* The meeting will be held at National Institute of Health (NIH). The address and room number is below: National Institute of Health, Natcher Conference Center, 45 Center Drive, Bethesda, MD 20892.

April 25—Balcony B.
April 26—Room E1/E2.

Security on the NIH Campus

All non-NIH employees are required to provide picture IDs upon entering the campus whether walking on to campus or driving on to campus, and all belongings are subject to searches. Increased security procedures are in place at all entrances to the NIH campus, including drive-in and walk-in access gates. Please allow adequate time when making your plans to attend the conference functions at the Natcher Conference Center. Preregistration will expedite the security process. Visitor parking is extremely limited and driving to the NIH campus for this event is not recommended.

Metrorail Service and Map

The NIH Campus is very accessible by the Washington D.C. area Metrorail (Metro) system. The Natcher Conference Center (Building 45) is located a short walk from the Medical Center Metro stop located on the Red Line. Note the signs and directions to the gated campus security entrance located behind the metro stop. For more details about the Washington DC area Metrorail services