

Dated: March 16, 2006.

**Andrew L. Bates,**

*Advisory Committee Management Officer.*

[FR Doc. E6-4193 Filed 3-22-06; 8:45 am]

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## NUCLEAR REGULATORY COMMISSION

### Advisory Committee on Reactor Safeguards Meeting of the Subcommittee on Plant License Renewal; Notice of Meeting

The ACRS Subcommittee on Plant License Renewal will hold a meeting on April 5, 2006, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland. The entire meeting will be open to public attendance. The agenda for the subject meeting shall be as follows:

**Wednesday, April 5, 2006—8:30 a.m.—12 Noon**

The purpose of this meeting is to discuss the License Renewal Application for Nine Mile Point and the related Safety Evaluation Report (SER) with open items prepared by the NRR staff. The Subcommittee will hear presentations by and hold discussions with representatives of the NRC staff, Constellation Energy Group, and other interested persons regarding this matter. The Subcommittee will gather information, analyze relevant issues and facts, and formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Members of the public desiring to provide oral statements and/or written comments should notify the Designated Federal Official, Mr. John G. Lamb (telephone 301/415-6855) five days prior to the meeting, if possible, so that appropriate arrangements can be made. Electronic recordings will be permitted.

Further information regarding this meeting can be obtained by contacting the Designated Federal Official between 7:30 a.m. and 4:15 p.m. (ET). Persons planning to attend this meeting are urged to contact the above named individual at least two working days prior to the meeting to be advised of any potential changes to the agenda.

Dated: March 16, 2006.

**Michael R. Snodderly,**

*Acting Branch Chief, ACRS/ACNW.*

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## NUCLEAR REGULATORY COMMISSION

### Notice of Availability of Model Application Concerning Technical Specifications for Boiling Water Reactor Plants to Risk-Inform Requirements Regarding Selected Required Action End States Using the Consolidated Line Item Improvement Process

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Notice of availability.

**SUMMARY:** Notice is hereby given that the staff of the Nuclear Regulatory Commission (NRC) has prepared a model application related to the revision of Boiling Water Reactor (BWR) plant required action end state requirements in technical specifications (TS). The purpose of this model is to permit the NRC to efficiently process amendments that propose to revise BWR TS required action end state requirements. Licensees of nuclear power reactors to which the model applies may request amendments utilizing the model application.

**DATES:** The NRC staff issued a **Federal Register** Notice (70 FR 74037, December 14, 2005) that provided a model safety evaluation (SE) and a model no significant hazards consideration (NSHC) determination relating to changing BWR TS required action end state requirements. The NRC staff hereby announces that the model SE and NSHC determination may be referenced in plant-specific applications to adopt the changes. The staff has posted a model application on the NRC Web site to assist licensees in using the consolidated line item improvement process (CLIP) to revise the BWR TS required action end state requirements. The NRC staff can most efficiently consider applications based upon the model application if the application is submitted within a year of this **Federal Register** Notice.

**FOR FURTHER INFORMATION CONTACT:** T. R. Tjader, Mail Stop: O12H2, Division of Inspection and Regional Support, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, telephone 301-415-1187.

#### SUPPLEMENTARY INFORMATION:

##### Background

Regulatory Issue Summary 2000-06, "Consolidated Line Item Improvement Process for Adopting Standard Technical Specification Changes for Power Reactors," was issued on March 20, 2000. The Consolidated Line Item

Improvement Process (CLIP) is intended to improve the efficiency of NRC licensing processes. This is accomplished by processing changes to the standard TS (STS) in a manner that supports subsequent license amendment applications. The CLIP includes an opportunity for the public to comment on proposed changes to the STS following a preliminary safety assessment by the NRC staff and finding that the change will likely be offered for adoption by licensees. The CLIP includes NRC staff evaluation of any comments received for a proposed change to the STS, and either a reconsideration of the change or an announcement of the availability of the change for adoption by licensees. Those licensees opting to apply for the subject change to their TS are responsible for reviewing the staff's evaluation, referencing the applicable technical justifications, and providing any necessary plant-specific information. Each amendment application made in response to the notice of availability will be processed and noticed in accordance with applicable rules and NRC procedures.

This notice involves the revision of BWR TS required action end state requirements. This change was proposed for incorporation into the STS by participants in the Owners Groups Technical Specification Task Force (TSTF) and is designated TSTF-423, Revision 0. TSTF-423, as well as the NRC staff's safety evaluation and model application, may be examined, and/or copied for a fee, at the NRC's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records are accessible electronically from the ADAMS Public Library component on the NRC Web site, (the Electronic Reading Room). TSTF-423, the NRC staff's safety evaluation, and the model application, can be viewed on the NRC Web site at: (<http://www.nrc.gov/reactors/operating/licensing/techspecs.html>).

##### Applicability

This proposal to modify technical specification requirements by the adoption of TSTF-423 is applicable to all licensees of BWR plants who have adopted or will adopt, in conjunction with the change, technical specification requirements for a Bases control program consistent with the TS Bases Control Program described in Section 5.5 of the BWR STS. Licensees that have not adopted requirements for a Bases control program by converting to the improved STS or by other means, are requested to include the requirements