

Dated: October 28, 2004.

Michael J. Kurtz,

*Assistant Archivist for Records Services—
Washington, DC.*

[FR Doc. 04-24501 Filed 11-2-04; 8:45 am]

BILLING CODE 7515-01-P

NATIONAL SCIENCE FOUNDATION

Notice of Intent To Seek Approval to Revise and Extend an Information Collection

AGENCY: National Science Foundation.

ACTION: Notice and request for comments.

SUMMARY: The National Science Foundation (NSF) is announcing plans to request clearance of this collection. In accordance with the requirement of section 3506(c)(2)(A) of the Paperwork Reduction Act of 1995 (Pub. L. 104-13), we are providing opportunity for public comment on this action. After obtaining and considering public comment, NSF will prepare the submission requesting that OMB approve clearance of this collection for no longer than 1 year.

DATES: Written comments on this notice must be received by January 3, 2005, to be assured of consideration. Comments received after that date will be considered to the extent practicable.

FOR FURTHER INFORMATION CONTACT: Contact Suzanne H. Plimpton, Reports Clearance Officer, National Science Foundation, 4201 Wilson Boulevard, Suite 295, Arlington, VA 22230; telephone (703) 292-7556; or send e-mail to splimpto@nsf.gov. Individuals who use a telecommunications device for the deaf (TDD) may call the Federal Information Relay Service (FIRS) at 1 (800) 877-8339 between 8 a.m. and 8 p.m., Eastern time, Monday through Friday. You also may obtain a copy of the data collection instrument and instructions from Ms. Plimpton.

SUPPLEMENTARY INFORMATION:

Title of Collection: Follow-up Research on Undergraduate Research Opportunities (URO-2).

OMB Number: 3145-0121.

Expiration Date of Approval: April 30, 2006.

Type of request: Intent to seek approval to revise and extend an information collection for one year.

Abstract: Follow-up Research on Undergraduate Research Opportunities (URO-2).

Proposed Project: The National Science Foundation (NSF) manages a number of programs that provide meaningful research experiences for undergraduate students. This suite of

programs includes: Research Experiences for Undergraduates (REU), both the Site and Supplement components; Research in Undergraduate Institutions (RUI); the undergraduate research components in several of NSF's large research centers programs, e.g., Engineering Research Centers (ERC) Programs, Science and Technology Centers (STCs); and several institution-wide resources development programs in which undergraduate research experiences are often one component.

These Programs provide a wide range of US undergraduate students with opportunities to conduct hands-on research under the mentorship of graduate students, postdoctoral fellows, and faculty in various types of higher education institutions, including small liberal arts colleges, minority-serving institutions, research universities, as well as non-profit institutions in which science or engineering research is conducted.

The purpose of the proposed evaluation is to follow-up on undergraduate participants in research experiences supported by NSF who were surveyed in 2003. The 2003 survey collected information about why participants chose to participate in research, the nature of the research activities, effects of research on participants knowledge, skills, confidence, awareness, and academic and career interests and aspirations. The proposed survey will provide information about participants' current academic and employment status (in 2003, most of the respondents were in their senior year of college) and participants' current perceptions of the effects of their undergraduate research experiences on their career and academic decisions. The survey database will be linked to that of the 2003 survey to assess differences on a number of dimensions, including NSF program, academic major, type of academic institution, and sex and race/ethnicity of the participant.

Use of the information: NSF and others who design undergraduate research programs will be able to use the information to help design programs that meet the needs of different kinds of students in different kinds of settings.

Estimate of Burden: Public reporting burden for this collection of information is estimated to average 30 minutes per response.

Respondents: Individuals.

Estimated Number of Respondents: 2900.

Estimated Total Annual Burden on Respondents: 1450 hours—2900 respondents at 30 minutes each.

Frequency of response: One time.

Comments: Comments are invited on (a) whether the proposed collection of information is necessary for the proper performance of the functions of the Agency, including whether the information shall have practical utility; (b) the accuracy of the Agency's estimate of the burden of the proposed collection of information; (c) ways to enhance the quality, utility, and clarity of the information on respondents; and (d) ways to minimize the burden of the collection of information on those who are to respond.

Dated: October 29, 2004.

Suzanne H. Plimpton,

Reports Clearance Officer, National Science Foundation.

[FR Doc. 04-24533 Filed 11-2-04; 8:45 am]

BILLING CODE 7555-01-M

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-498 and 50-499]

STP Nuclear Operating Company; South Texas Project, Unit No. 1 and 2; Exemption

1.0 Background

The STP Nuclear Operating Company (STPNOC or the licensee) is the holder of Facility Operating License Nos. NPF-76 and NPF-80, which authorize operation of South Texas Project (STP), Units 1 and 2, respectively. The licenses provide, among other things, that the facility is subject to all rules, regulations, and orders of the Nuclear Regulatory Commission (NRC, the Commission) now or hereafter in effect.

The facility consists of two pressurized water reactors located in Matagorda County, Texas.

2.0 Request/Action

Title 10 of the Code of Federal Regulations (10 CFR) part 50, section 50.44, specifies requirements for the control of hydrogen gas generated after a postulated loss-of-coolant accident (LOCA). Section 50.46 of 10 CFR contains acceptance criteria for the emergency core cooling system (ECCS) for reactors with zircaloy or ZIRLO™ clad fuel. Appendix K to 10 CFR part 50 requires, among other things, that the Baker-Just equation be used to predict the rates of energy release, hydrogen concentration, and cladding oxidation from the metal-water reaction. Of these three regulations (10 CFR 50.44, 50.46, and Appendix K to 10 CFR part 50), 10 CFR 50.44 is the only one that has undergone considerable changes relative to its previous version, changes that became effective on January 1, 2004.

Prior to that date, 10 CFR 50.44 specified requirements for the control of hydrogen gas generated after a postulated LOCA for reactors with zircaloy or ZIRLO™ clad fuel. The new regulation in 10 CFR 50.44 no longer identifies zircaloy or ZIRLO™ as requisite fuel cladding, nor does it identify the LOCA or 10 CFR 50.46 as bases. Because the intent of this exemption request relates solely to the specific types of cladding material specified in these regulations, no exemption is needed from the requirements of 10 CFR 50.44. As written, zircaloy or ZIRLO™ cladding continues to be the requisite fuel cladding that is explicitly identified in 10 CFR 50.46 and Appendix K to 10 CFR part 50. Therefore, an exemption from the requirements of 10 CFR 50.46 and Appendix K to 10 CFR part 50 is needed in order to irradiate up to eight lead test assemblies (LTAs) comprised of low tin (Optimized) ZIRLO™ at the STP, Units 1 and/or 2.

In summary, in a letter dated May 27, 2004 (Reference 1)¹, as supplemented by letter dated August 23, 2004 (Reference 2)², STPNOC requested an exemption from 10 CFR 50.44, "Standards for Combustible Gas Control System in Light-Water-Cooled Power Reactors"; 10 CFR 50.46, "Acceptance Criteria for Emergency Core Cooling Systems for Light-Water Nuclear Power Reactors"; and Appendix K to 10 CFR part 50, "ECCS Evaluation Models," which would allow irradiation of up to eight LTAs containing fuel rods, guide tubes, and instrumentation tubes fabricated with Optimized ZIRLO™. Optimized ZIRLO™ is not within the licensing basis of the approved ZIRLO™ as described in WCAP-12610-P-A (Reference 3)³ for STP, Units 1 and 2. Irradiation of up to eight Optimized ZIRLO™ LTAs in STP Units 1 and/or 2 will provide data on fuel and material performance to support future licensing activities.

¹ Letter from T.J. Jordan (South Texas) to U.S. Nuclear Regulatory Commission, "The South Texas Project, Units 1 and 2 Request for Exemption Pursuant to 10 CFR 50.12 Exemption to the Fuel Cladding Material Specified in 10 CFR 50.44, 10 CFR 50.46, and 10 CFR 50 Appendix K," Docket Nos. STN 50-498 and STN 50-499, May 27, 2004, ADAMS Accession No. ML041590200.

² Letter from T.J. Jordan (South Texas) to U.S. Nuclear Regulatory Commission, "The South Texas Project, Units 1 and 2 Response to Request for Additional Information Regarding Exemption to Use a Low Tin Cladding," Docket Nos. STN 50-498 and STN 50-499, August 23, 2004, ADAMS Accession No. ML042430272.

³ Westinghouse Electric Company Topical Report, WCAP-12610-P-A, "VANTAGE+ Fuel Assembly Reference Core Report," April 1995.

3.0 Discussion

The staff has previously reviewed exemption requests for LTA programs comprised of fuel with Optimized ZIRLO™ cladding material manufactured by Westinghouse Electric Company (Westinghouse). Exemptions from 10 CFR 50.46 and Appendix K to 10 CFR part 50 for use of Optimized ZIRLO™ have been issued by the NRC staff for Millstone, Unit 3 (Reference 4)⁴, Catawba Station (Reference 5)⁵, and Calvert Cliffs, Unit 2 (Reference 6)⁶.

3.1 Material Evaluation

3.1.1 Fuel Mechanical Design

Tin is a solid solution strengthener and α -phase stabilizer present entirely in the base α -phase zirconium crystalline structure. Potential impacts of a reduced tin content on material properties include (1) a reduced tensile strength, (2) an increased thermal creep rate, (3) an increased irradiation growth rate, (4) a reduced $\alpha \leftrightarrow \alpha + \beta$ phase transition temperature, and (5) an improved corrosion resistance. The slight reduction in tin content will not affect the size, shape, or distribution of any second phase or inter-metallic precipitates nor the overall microstructure of this developmental zirconium alloy. With a consistent microstructure, Optimized ZIRLO™ will exhibit material characteristics very similar to that of ZIRLO™.

In Reference 2, the licensee provided information concerning their post-irradiation examination plan. In Reference 2, the licensee stated that their plan would be consistent with those of the other Optimized ZIRLO™ irradiation programs currently underway. As with the post-irradiation examinations involved in the other irradiation programs, the detailed examinations in the licensee's Optimized ZIRLO™ irradiation program will be based on the fuel duty, cycle performance, need for specific information, and time available on site during refueling outages. The measured

⁴ Letter from U.S. Nuclear Regulatory Commission to D.A. Christian (Dominion), "Millstone Power Station, Unit No. 3, Exemption from the Requirements of Title 10 of the Code of Federal Regulations (10 CFR) Part 50.44, 10 CFR 50.46, and 10 CFR part 50, Appendix K," Docket No. 50-423, February 11, 2004, ADAMS Accession No. ML040070238.

⁵ Letter from U.S. Nuclear Regulatory Commission to D.M. Jamil (Duke), "Catawba Nuclear Station, Units 1 and 2 RE: Exemption from the Requirements of 10 CFR 50.44, 10 CFR 50.46, and 10 CFR 50 Appendix K," August 4, 2003, ADAMS Accession No. ML032060473.

⁶ Letter from U.S. Nuclear Regulatory Commission to P.E. Katz (Constellation), "Calvert Cliffs Nuclear Power Plant, Unit No. 2," December 3, 2002, ADAMS Accession No. ML022540002.

parameters will include rod profilometry, rod wear, assembly and rod growth, assembly bow, grid cell dimensions, and oxide thickness. As a result of these post-irradiation examinations, any negative aspects of the Optimized ZIRLO™ performance, including the potential impacts of reduced tin content identified above, will be identified and resolved. Furthermore, significant deviations from model predictions will be reconciled.

The fuel rod burnup and fuel duty experienced by the Optimized ZIRLO™ LTAs in STP, Units 1 and 2, will remain well within the operating experience base and applicable licensed limits for ZIRLO™.

Utilizing currently approved fuel performance and fuel mechanical design models and methods, the STP, Units 1 and 2, and Westinghouse will perform cycle-specific reload evaluations to ensure that the Optimized ZIRLO™ LTAs satisfy design criteria.

Based upon the irradiation experience of LTAs with ZIRLO™ of a similar low tin content, expected performance due to similar material properties, and an extensive LTA post-irradiation examination program aimed at qualifying model predictions, the NRC staff finds the Optimized ZIRLO™ LTA mechanical design acceptable for STP, Units 1 and 2.

3.1.2 Core Physics and Non-LOCA Safety Analysis

The STP, Units 1 and 2, exemption request relates solely to the specific types of cladding material specified in the regulations. Due to similar material properties, any impact of Optimized ZIRLO™ on the safety analysis models and methods is expected to be minimal. Utilizing currently approved core physics, core thermal-hydraulics, and non-LOCA safety analysis models and methods, the licensee and Westinghouse will perform cycle-specific reload evaluations to ensure that the LTAs satisfy design criteria. Fuel management guidelines will require that LTAs be placed in non-limiting core locations. In Reference 2, the licensee described how the power peaking margin would be used to ensure that LTAs will not be limiting.

Based upon the use of approved models and methods, expected material performance, and the placement of LTAs in non-limiting core locations, the NRC staff finds that the irradiation of up to eight Optimized ZIRLO™ LTAs in STP, Units 1 and 2, will not result in unsafe operation nor violation of specified acceptable fuel design limits. Furthermore, in the event of a design-basis accident, these LTAs will not

promote consequences beyond those currently analyzed.

3.2 ECCS Performance and Exemptions

Pursuant to 10 CFR 50.12, the Commission may, upon application by any interested person or own initiative, grant exemptions from the requirements of 10 CFR part 50 when (1) the exemptions are authorized by law, will not present an undue risk to public health or safety, and are consistent with the common defense and security; and (2) special circumstances are present. Special circumstances are present if application of the regulation in the particular circumstances would not serve the underlying purpose of the rule, or is not necessary to achieve the underlying purpose of the rule.

The underlying purpose of 10 CFR 50.46 is to establish acceptance criteria for ECCS performance. In Addendum 1 to WCAP-12610-P-A (Reference 7)⁷, Westinghouse demonstrates that the material properties of Optimized ZIRLO™ are similar to those of the currently approved ZIRLO™ cladding and that the ECCS acceptance criteria for ZIRLO™ clad fuel are also applicable to fuel with Optimized ZIRLO™ cladding. Ring compression tests performed by Westinghouse on Optimized ZIRLO™ demonstrate an acceptable retention of ductility up to 10 CFR 50.46 limits of 2200 °F peak cladding temperature and 17 percent total oxidation. Utilizing currently approved LOCA models and methods, Westinghouse will perform cycle-specific reload evaluations to ensure that the Optimized ZIRLO™ LTAs satisfy 10 CFR 50.46 acceptance criteria.

Paragraph I.A.5 of Appendix K to 10 CFR part 50 states that the rates of energy, hydrogen concentration, and cladding oxidation from the metal-water reaction shall be calculated using the Baker-Just equation. Since the Baker-Just equation presumes the use of zircaloy clad fuel, strict application of the rule would not permit use of the equation for the Optimized ZIRLO™ LTA cladding for determining acceptable fuel performance. Metal-water reaction tests performed by Westinghouse on Optimized ZIRLO™ (documented in Appendix B of Addendum 1 to WCAP-12610-P-A) demonstrate conservative reaction rates relative to the Baker-Just equation. Thus, application of Appendix K, Paragraph I.A.5, in these circumstances, is not necessary for the licensee to

achieve the underlying purpose of the regulation.

Based upon the results of metal-water reaction tests and ring-compression tests, which ensure the applicability of ECCS models and acceptance criteria and the use of approved LOCA models to ensure that the Optimized ZIRLO™ LTAs satisfy 10 CFR 50.46 acceptance criteria, the NRC staff finds it acceptable to grant an exemption from the requirements of 10 CFR 50.46 and Appendix K to 10 CFR part 50 for the use of up to eight LTAs in STP, Units 1 and 2.

3.3 Special Circumstances

In summary, the NRC staff has reviewed the licensee's request for an exemption to allow up to eight LTAs containing fuel rods, guide thimble tubes, and instrumentation tubes fabricated with Optimized ZIRLO™ to be used in STP, Units 1 and 2. Based on the NRC staff's evaluation, as set forth above, the NRC staff considers that granting the proposed exemption will not defeat the underlying purpose of 10 CFR 50.46, or Appendix K to 10 CFR part 50. Accordingly, special circumstances, are present pursuant to 10 CFR 50.12(a)(2)(ii).

3.4 Other Standards in 10 CFR 50.12

The NRC staff reviewed information provided by the licensee in References 1 and 2 to support the exemption request, and concluded that the use of Optimized ZIRLO™ would satisfy 10 CFR 50.12(a) as follows:

(1) The requested exemption is authorized by law:

No law precludes the activities covered by this exemption request. The Commission, based on technical reasons set forth in rulemaking records, specified the specific cladding materials identified in 10 CFR 50.46 and 10 CFR part 50, Appendix K. Cladding materials are not specified by statute.

(2) The requested exemption does not present an undue risk to the public health and safety. As stated by the licensee in Reference 1:

The lead test assembly safety evaluation will ensure that these acceptance criteria are met following insertion of the assemblies containing Optimized ZIRLO™ material. Fuel assemblies using Optimized ZIRLO™ cladding will be evaluated using NRC-approved analytical methods and will address the changes in the cladding material properties. The safety analysis for the South Texas Project is supported by the applicable technical specifications. The South Texas Project reload cores containing Optimized ZIRLO™ cladding will continue to be operated in accordance with the operating limits specified in the technical specifications. Lead test assemblies using

Optimized ZIRLO™ cladding will be placed in non-limiting core locations. Therefore, this exemption will not pose an undue risk to public health and safety.

The NRC staff has evaluated these considerations as set forth in Section 3.1 and 3.2 of this Exemption. For the reasons set forth in Sections 3.1 and 3.2, the NRC staff concludes that Optimized ZIRLO™ may be used as a cladding material for up to eight LTAs to be placed in non-limiting core locations in STP, Units 1 and 2, and that an exemption from the requirements of 10 CFR 50.46 and 10 CFR part 50, Appendix K, does not pose an undue risk to the public health and safety.

(3) The common defense and security are not affected and, therefore, not endangered by this exemption.

4.0 Conclusion

Accordingly, the Commission has determined that, pursuant to 10 CFR 50.12(a), the exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with the common defense and security. Also, special circumstances are present. Therefore, the Commission hereby grants STPNOC an exemption from the requirements of 10 CFR part 50, Appendix K and Section 50.46, for the use of up to eight LTAs containing Optimized ZIRLO™ in STP, Units 1 and 2, up to a lead rod average burnup of 62,000 megawatt days per metric ton of uranium.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will not have a significant effect on the quality of the human environment (69 FR 45352).

This exemption is effective upon issuance.

For the Nuclear Regulatory Commission.

Ledyard B. Marsh,

Director, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 04-24493 Filed 11-2-04; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Draft Appendix C (DG-1138) to Regulatory Guide 1.200, "An Approach for Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities," Workshop

The U.S. Nuclear Regulatory Commission (NRC) has issued draft Appendix C, "NRC Staff Position on ANS External Hazards PRA Standard" in August 2004. This Appendix will be

⁷ Westinghouse Electric Company Topical Report, Addendum 1 to WCAP-12610-P-A and CENPD-404-P-A, "Optimized ZIRLO", February 2003.