

Amendment Nos.: Unit 1—171; Unit 2—172.

Facility Operating License Nos. DPR-80 and DPR-82: The amendments revised the Technical Specifications.

Date of initial notice in Federal Register: December 24, 2002 (67 FR 78522)

The November 21, 2003, and March 9, 2004, supplemental letters provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination.

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated July 2, 2004.

No significant hazards consideration comments received: No.

South Carolina Electric & Gas Company, South Carolina Public Service Authority, Docket No. 50-395, Virgil C. Summer Nuclear Station, Unit No. 1, Fairfield County, South Carolina

Date of application for amendment: September 19, 2003.

Brief description of amendment: This amendment revised Surveillance Requirement 4.2.4.2 to specifically identify the Power Distribution Monitoring System being used in determining the Quadrant Power Tilt Ratio with one inoperable Power Range Channel.

Date of issuance: July 6, 2004.

Effective date: As of the date of issuance and shall be implemented within 30 days from the date of issuance.

Amendment No.: 168.

Renewed Facility Operating License No. NPF-12: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: March 30, 2004 (69 FR 16623).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated July 6, 2004.

No significant hazards consideration comments received: No.

Tennessee Valley Authority, Docket No. 50-327, Sequoyah Nuclear Plant, Units 1 and 2, Hamilton County, Tennessee

Date of application for amendment: March 5, 2004.

Brief description of amendment: The amendment revises the reactor coolant pump flywheel inspection interval from 10 years to 20 years.

Date of issuance: July 8, 2004.

Effective date: As of the date of issuance and shall be implemented within 45 days of issuance.

Amendment Nos.: 293 and 283.

Facility Operating License No. DPR-77 and DPR-79: Amendment revises the technical specifications.

Date of initial notice in Federal Register: April 13, 2004 (69 FR 19577).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated July 8, 2004.

No significant hazards consideration comments received: No.

Union Electric Company, Docket No. 50-483, Callaway Plant, Unit 1, Callaway County, Missouri

Date of application for amendment: April 8, 2004.

Brief description of amendment: The amendment revises TS 5.5.7, "Reactor Coolant Pump Flywheel Inspection Program," to increase the inspection interval from 10 years to 20 years.

Date of issuance: July 12, 2004.

Effective date: July 12, 2004, and shall be implemented within 90 days from the date of issuance.

Amendment No.: 163.

Facility Operating License No. NPF-30: The amendment revised the Technical Specifications.

Date of initial notice in Federal Register: May 11, 2004 (69 FR 26193).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated July 12, 2004.

No significant hazards consideration comments received: No.

Wolf Creek Nuclear Operating Corporation, Docket No. 50-482, Wolf Creek Generating Station, Coffey County, Kansas

Date of amendment request: April 30, 2003, as supplemented by letters dated December 18, 2003, and April 13, 2004.

Brief description of amendment: The amendment revises several surveillance requirements (SRs) in Technical Specification (TS) 3.8.1 on alternating current sources for plant operation. The revised SRs have notes deleted or modified to allow the SRs to be performed, or partially performed, in reactor modes that previously were not allowed by the TSs. The proposed changes to SRs 3.8.4.7 and 3.8.4.8 for direct current sources were withdrawn by letter dated April 13, 2004.

Date of issuance: July 12, 2004.

Effective date: July 12, 2004, and shall be implemented within 90 days of the date of issuance including the incorporation of the changes to the TS Bases for TS 3.8.1 as described in the licensee's letters dated April 30 and December 18, 2003, and April 13, 2004.

Amendment No.: 154.

Facility Operating License No. NPF-42: The amendment revised the Technical Specifications.

Date of initial notice in Federal Register: June 10, 2003 (68 FR 34673).

The December 18, 2003, and April 13, 2004, supplemental letters provided additional clarifying information, did not expand the scope of the application as noticed and did not change the staff's original proposed no significant hazards consideration determination. The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated July 12, 2004.

No significant hazards consideration comments received: No.

Dated at Rockville, Maryland, this 26th day of July 2004.

For the Nuclear Regulatory Commission.

James E. Lyons,

Deputy Director, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

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NUCLEAR REGULATORY COMMISSION

Draft Regulatory Guide; Issuance, Availability

The Nuclear Regulatory Commission has issued for public comment a proposed revision of a guide in its Regulatory Guide Series. Regulatory Guides are developed to describe and make available to the public such information as methods acceptable to the NRC for implementing specific parts of the NRC's regulations, techniques used by the staff in evaluating specific problems or postulated accidents, and data needed by the staff in its review of applications for permits and licenses.

The draft guide is temporarily identified by its task number, DG-1124, which should be mentioned in all correspondence concerning this draft guide. Draft regulatory guide DG-1124, "Design, Fabrication, and Materials Code Case Acceptability, ASME Section III," is proposed Revision 33 of Regulatory Guide 1.84. The regulation in 10 CFR 50.55a(c), "Reactor Coolant Pressure Boundary," requires, in part, that components of the reactor coolant pressure boundary must be designed, fabricated, erected, and tested in accordance with the requirements for Class 1 components of Section III, "Rules for Construction of Nuclear Power Plant Components," of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel (B&PV) Code or equivalent quality standards. The ASME publishes a new edition of the B&PV Code, which includes Section III, every three years, and new addenda every year. The latest

editions and addenda of Section III that have been approved for use by the NRC are referenced in 10 CFR 50.55a(b). The ASME also publishes Code cases quarterly. Code cases provide alternatives developed and approved by ASME to existing Code requirements. This draft regulatory guide identifies the Code cases that have been determined by the NRC to be acceptable alternatives to applicable parts of Section III. Section III Code cases not yet endorsed by the NRC may be implemented through 10 CFR 50.55a(a)(3), which permits the use of alternatives to the Code requirements referenced in 10 CFR 50.55a provided that the proposed alternatives result in an acceptable level of quality and safety, and that their use is authorized by the Director of the Office of Nuclear Reactor Regulation.

This is a draft guide and does not represent an official NRC staff position. Because Code cases approved by the NRC in a final guide may be used voluntarily by licensees as an alternative to compliance with ASME Code provisions, the final guide will be incorporated by reference into 10 CFR 50.55a through rulemaking.

Comments may be accompanied by relevant information or supporting data. Written comments may be submitted by mail to the Rules and Directives Branch, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; or they may be hand-delivered to the Rules and Directives Branch, Office of Administration, at 11555 Rockville Pike, Rockville, MD. Copies of comments received may be examined at the NRC's Public Document Room, 11555 Rockville Pike, Rockville, MD. Comments will be most helpful if received by September 2, 2004.

You may also provide comments via the NRC's interactive rulemaking web site through the NRC home page (<http://www.nrc.gov>). This site provides the ability to upload comments as files (any format) if your web browser supports that function. For information about the interactive rulemaking web site, contact Ms. Carol Gallagher, (301) 415-5905; e-mail cag@nrc.gov. For technical information about Draft Regulatory Guide DG-1124, contact Mr. W.E. Norris at (301) 415-6796 (e-mail wen@nrc.gov).

Although a deadline is given for comments on these draft guides, comments and suggestions in connection with items for inclusion in guides currently being developed or improvements in all published guides are encouraged at any time.

Regulatory guides are available for inspection at the NRC's Public Document Room, 11555 Rockville Pike, Rockville, MD; the PDR's mailing

address is USNRC PDR, Washington, DC 20555-0001; telephone (301) 415-4737 or (800) 397-4209; fax (301) 415-3548; e-mail pdr@nrc.gov. Requests for single copies of draft or final regulatory guides (which may be reproduced) or placement on an automatic distribution list for single copies of future draft guides in specific divisions should be made in writing to the U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Reproduction and Distribution Services Section, or by fax to (301) 415-2289; e-mail distribution@nrc.gov. Telephone requests cannot be accommodated. Regulatory guides are not copyrighted, and NRC approval is not required to reproduce them. (5 U.S.C. 552(a))

Dated at Rockville, Maryland this 20th day of April, 2004.

For the Nuclear Regulatory Commission.

Michael E. Mayfield,

*Director, Division of Engineering Technology,
Office of Nuclear Regulatory Research.*

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NUCLEAR REGULATORY COMMISSION

Draft Regulatory Guide; Issuance, Availability

The Nuclear Regulatory Commission (NRC) has issued for public comment a proposed revision of a guide in its Regulatory Guide Series. Regulatory Guides are developed to describe and make available to the public such information as methods acceptable to the NRC staff for implementing specific parts of the NRC's regulations, techniques used by the staff in evaluating specific problems or postulated accidents, and data needed by the staff in its review of applications for permits and licenses.

The draft guide is temporarily identified by its task number, DG-1125, which should be mentioned in all correspondence concerning this draft guide. Draft Regulatory Guide DG-1125, "Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1," is proposed Revision 14 of Regulatory Guide 1.147. The regulation at 10 CFR 50.55a(g), "Inservice Inspection Requirements," requires, in part, that Classes 1, 2, 3, MC, and CC Components and their supports meet the requirements of Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel (B&PV) Code or equivalent quality standards. Every 3 years the

ASME publishes a new edition of the B&PV Code, including Section XI, and new addenda are published every year. The latest editions and addenda of Section XI that have been approved for use by the NRC are referenced in 10 CFR 50.55a(b). The ASME also publishes Code cases quarterly. Code cases provide alternatives to existing Code requirements that were developed and approved by the ASME. This regulatory guide identifies the Code cases that have been determined by the NRC to be acceptable alternatives to applicable parts of Section XI. These Code cases may be used by licensees without a request for authorization from the NRC provided that they are used with any identified limitations or modifications. Section XI Code cases not yet endorsed by the NRC may be implemented through 10 CFR 50.55a(a)(3), which permits the use of alternatives to the Code requirements referenced in 10 CFR 50.55a provided that the proposed alternatives result in an acceptable level of quality and safety and that their use is authorized by the Director of the Office of Nuclear Reactor Regulation.

This draft guide has not received complete staff approval and does not represent an official NRC staff position. Because Code cases approved by the NRC in a final guide may be used voluntarily by licensees as an alternative to compliance with ASME Code provisions, the final guide will be incorporated by reference into 10 CFR 50.55a through rulemaking.

A document entitled "Evaluation of Code Cases" is attached to the proposed rulemaking associated with the draft guide. The document provides a basis for each condition in the draft guide. Public comments are encouraged on the Code case conditions. It should be noted that Code Cases N-416-3 and N-504-2 are listed in the draft guide as unconditionally acceptable. The NRC is proposing to condition Code Case N-416-3 in response to a recent licensee submittal. The NRC does not believe that the application of the Code case as described in the submittal would provide adequate assurance of component structural integrity. A condition is also being proposed for Code Case N-504-2. The American Society of Mechanical Engineers (ASME) recently addressed a revision to Code Case N-504-2. The NRC is proposing to condition the use of Code Case N-504-2 based on this recent ASME action. The proposed conditions are discussed in Section 4.7 of the "Evaluation of Code Cases." Because the industry actions occurred after the draft guide had been published but prior to release of the guide for public comment,