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SUPPLEMENTARY INFORMATION:

I. Introduction

The U.S. Nuclear Regulatory Commission (NRC) is considering the issuance of a license amendment to Adolor Corporation for Materials License No. 37-30369-01, to authorize release of its facility in Malvern, Pennsylvania for unrestricted use and has prepared an Environmental Assessment (EA) in support of this action in accordance with the requirements of 10 CFR part 51. Based on the EA, the NRC has concluded that a Finding of No Significant Impact (FONSI) is appropriate.

II. EA Summary

The purpose of the proposed action is to allow for the release of the licensee's Malvern, Pennsylvania facility for unrestricted use. Adolor Corporation was authorized by NRC from August 20, 1997 to use radioactive materials for research and development purposes at the site. On July 18, 2003, Adolor Corporation requested that NRC release the facility for unrestricted use. Adolor Corporation has conducted surveys of the facility and determined that the facility meets the license termination criteria in subpart E of 10 CFR part 20.

III. Finding of No Significant Impact

The NRC staff has evaluated Adolor Corporation's request and the results of the surveys and has concluded that the completed action complies with 10 CFR part 20. The staff has prepared the EA (summarized above) in support of the proposed license amendment to terminate the license and release the facility for unrestricted use. On the basis of the EA, the NRC has concluded that the environmental impacts from the proposed action are expected to be insignificant and has determined not to prepare an environmental impact statement for the proposed action.

IV. Further Information

The EA and the documents related to this proposed action, including the application for the license amendment and supporting documentation, are available for inspection at NRC's Public Electronic Reading Room at <http://www.nrc.gov/reading-rm/adams.html> (ADAMS Accession Nos. ML031880271, ML032030484, ML032541074 and ML040060259). These documents are also available for inspection and copying for a fee at the Region I Office, 475 Allendale Road, King of Prussia, Pennsylvania, 19406.

Dated at King of Prussia, Pennsylvania this 7th day of January, 2004.

For the Nuclear Regulatory Commission.

John D. Kinneman,

Chief, Nuclear Materials Safety Branch 2, Division of Nuclear Materials Safety, Region I.

[FR Doc. 04-787 Filed 1-13-04; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket No. 72-17]

Portland General Electric Company, Trojan Independent Spent Fuel Storage Installation; Notice of Docketing of Materials License SNM-2509 Amendment Application

By letter dated December 9, 2003, Portland General Electric Company (PGE) submitted an application to the Nuclear Regulatory Commission (NRC or the Commission), in accordance with 10 CFR part 72, requesting the amendment of the Trojan Independent Spent Fuel Storage Installation (ISFSI) license (SNM-2509) and the Technical Specifications for the ISFSI located at Columbia County, Oregon. PGE is seeking Commission approval to amend the materials license and the ISFSI Technical Specifications to reflect completion of dry storage cask loading operations and incorporation of an administrative change to the Trojan ISFSI Technical Specifications to conform to a recent NRC Final Rule, "Event Notification Requirements," (63 FR 33611 dated June 5, 2003).

This application was docketed under 10 CFR part 72; the ISFSI Docket No. is 72-17 and will remain the same for this action. The amendment of an ISFSI license is subject to the Commission's approval.

The Commission may issue either a notice of hearing or a notice of proposed action and opportunity for hearing in accordance with 10 CFR 72.46(b)(1) or, if a determination is made that the amendment does not present a genuine issue as to whether public health and safety will be significantly affected, take immediate action on the amendment in accordance with 10 CFR 72.46(b)(2) and provide notice of the action taken and an opportunity for interested persons to request a hearing on whether the action should be rescinded or modified.

For further details with respect to this amendment, see the application dated December 9, 2003, which is publically available in the records component of NRC's Agencywide Documents Access and Management System (ADAMS). The NRC maintains ADAMS, which

provides text and image files of NRC's public documents. These documents may be accessed through the NRC's Public Electronic Reading Room on the Internet at <http://www.nrc.gov/reading-rm/adams.html>. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1-800-397-4209, 301-415-4737 or by email to pdr@nrc.gov.

Dated at Rockville, Maryland, this 31st day of December, 2003.

For the Nuclear Regulatory Commission.

Christopher M. Regan,

Project Manager, Spent Fuel Project Office, Office of Nuclear Material Safety and Safeguards.

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-263]

Nuclear Management Company, LLC, Monticello Nuclear Generating Plant Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an amendment to Facility Operating License No. DPR-22, issued to Nuclear Management Company, LLC (NMC), for operation of the Monticello Nuclear Generating Plant (Monticello), located in Wright County, Minnesota. Therefore, as required by 10 CFR 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

The proposed action would revise the Monticello operating license to change the Monticello design bases and the Updated Safety Analysis Report (USAR). The proposed action would revise the existing analyses for the following:

- Long-term containment response to the design-basis loss-of-coolant accident (LOCA).
- Containment overpressure (the pressure above the initial containment pressure) required for adequate available net positive suction head (NPSH) for the low-pressure emergency core cooling system (ECCS) pumps following a LOCA.

NMC intends to use these analyses to justify restoring the service water temperature to its licensing-basis value of 90 degrees F. NMC administratively