

**MATTERS TO BE CONSIDERED:** 1. One (1) Creditor Claim Appeal. Closed pursuant to Exemptions (6) and (8).

**FOR FURTHER INFORMATION CONTACT:** Becky Baker, Secretary of the Board, Telephone: 703-518-6304.

**Becky Baker,**

*Secretary of the Board*

[FR Doc. 03-19305 Filed 7-24-03; 5:14 pm]

**BILLING CODE 7535-01-M**

## NATIONAL TRANSPORTATION SAFETY BOARD

### Sunshine Act Meeting

**TIME AND DATE:** 9:30 a.m., Tuesday, August 5, 2003.

**PLACE:** NTSB Conference Center, 429 L'Enfant Plaza, SW., Washington, D.C. 20594.

**STATUS:** The three items are Open to the Public.

**MATTERS TO BE CONSIDERED:**

7487A—Railroad Accident Report—Derailment of Amtrak Auto Train P052-18 on the CSXT Railroad near Crescent City, Florida, on April 18, 2002.

7575—Railroad Accident Report—Uncontrolled Movement, Collision and Passenger Fatality on the Angels Flight Railway in Los Angeles, California, on February 1, 2001.

7299A—Aviation Accident Report—Emery Worldwide Airlines, Inc., McDonnell Douglas DC-8-71F, N8079U, Rancho Cordova, California, on February 16, 2000.

News Media Contact: Telephone: (202) 314-6100.

Individuals requesting specific accommodations should contact Ms. Carolyn Dargan at (202) 314-6305 by Friday, August 1, 2003.

**FOR FURTHER INFORMATION CONTACT:** Vicky D'Onofrio, (202) 314-6410.

Dated: July 25, 2003.

**Vicky D'Onofrio,**

*Federal Register Liaison Officer.*

[FR Doc. 03-19324 Filed 7-25-03; 11:18 am]

**BILLING CODE 7533-01-M**

## NUCLEAR REGULATORY COMMISSION

[Docket No. 50-155 & 72-043]

### Consumers Energy Co., Big Rock Point Nuclear Plant; Notice of Receipt, Availability for Comment, and Meeting to Discuss License Termination Plan

The Nuclear Regulatory Commission (NRC) is in receipt of, and is making

available for public inspection and comment, the License Termination Plan (LTP) for the Big Rock Point Nuclear Facility (BRP) located in Charlevoix, Michigan.

Reactor operations at the BRP ended in August 29, 1997. The reactor was defueled and all fuel moved to an independent spent fuel storage installation (ISFSI) in March 2003. In accordance with NRC regulations in effect at that time, the licensee submitted a decommissioning plan for the BRP to the NRC in February 1995. When proposed amendments to the NRC's decommissioning regulations were published in the **Federal Register** on July 29, 1996 (61 FR 39278), the licensee requested that the review of the decommissioning plan be suspended. When the amended regulations became effective on August 28, 1996, the submitted decommissioning plan, as supplemented, became the BRP Post Shutdown Decommissioning Activities Report (PSDAR) pursuant to 10 CFR 50.82, as amended. A public meeting was held in Charlevoix, Michigan, on November 13, 1997, to provide information and gather public comment on the PSDAR. The facility is undergoing active decontamination and dismantlement.

In accordance with 10 CFR 50.82(a)(9), all power reactor licensees must submit an application for termination of their license. The application for termination of license must be accompanied by or preceded by an LTP submitted for NRC approval. If found acceptable by the NRC staff, the LTP is approved by license amendment, subject to such conditions and limitations as the NRC staff deems appropriate and necessary. The licensee submitted the proposed LTP for the BRP by application dated April 1, 2003. In accordance with 10 CFR 20.1405 and 10 CFR 50.82(a)(9)(iii), the NRC is providing notice to individuals in the vicinity of the site that the NRC is in receipt of the BRP LTP, and will accept comments from affected parties. In accordance with 10 CFR 50.82(a)(9)(iii), the NRC is also providing notice that the NRC staff will conduct a meeting to discuss the BRP LTP on Tuesday, August 5, 2003, at 7 p.m., at the Charlevoix Stroud Hall located at 12491 Waller Road, Charlevoix, Michigan 49720.

The BRP LTP and associated environmental report are available for public inspection at NRC's Public Document Room at NRC Headquarters, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852. These documents are available for public review through ADAMS, the NRC's

electronic reading room, at: <http://www.nrc.gov/reading-rm/adams.html>.

Dated at Rockville, Maryland, this 21st day of July, 2003.

For the Nuclear Regulatory Commission.

**Daniel M. Gillen,**

*Chief, Decommissioning Branch, Division of Waste Management, Office of Nuclear Material Safety and Safeguards.*

[FR Doc. 03-19215 Filed 7-28-03; 8:45 am]

**BILLING CODE 7590-01-P**

## NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-327 and 50-328]

### Tennessee Valley Authority; Sequoyah Nuclear Plant, Units 1 and 2, Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from title 10 of the Code of Federal Regulations (10 CFR) part 50, section 50.60 for Facility Operating License Nos. DPR-77 and DPR-79, issued to the Tennessee Valley Authority (TVA, the licensee), for operation of the Sequoyah Nuclear Plant (SQN), Units 1 and 2, located in Hamilton County, Tennessee. Therefore, as required by 10 CFR 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

#### Environmental Assessment

##### Identification of the Proposed Action

The proposed action would permit the use of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel (B&PV) Code, Section XI Code Case N-640, "Alternative Requirement Fracture Toughness for Development of P-T Limit Curves for ASME B&PV Code, Section XI, Code Case N-640," in lieu of 10 CFR 50, Appendix G, paragraph IV.A.2.b.

The regulation at 10 CFR part 50, section 50.60(a), requires, in part, that except where an exemption is granted by the Commission, all light-water nuclear power reactors must meet the fracture toughness requirements for the reactor coolant pressure boundary set forth in Appendix G to 10 CFR part 50. Appendix G of 10 CFR part 50 requires the establishment of pressure-temperature (P-T) limits for specific material fracture toughness requirements of the reactor coolant pressure boundary materials and mandates the use of the ASME B&PV Code, Section XI, Appendix G. The requirements in 10 CFR 50, Appendix