

Week of April 29, 2002—Tentative

Tuesday, April 30, 2002

9:30 a.m.

Discussion of Intergovernmental Issues
(Closed—Ex. 1)

Wednesday, May 1, 2002

8:55 a.m.

Affirmation Session (Public Meeting) (If
needed)

9:00 a.m.

Briefing on Results of Agency Action
Review Meeting—Reactors (Public
Meeting) (Contact: Robert Pascarella,
301-415-1245)This meeting will be webcast live at the
Web address—www.nrc.gov**Week of May 6, 2002—Tentative**There are no meetings scheduled for the
Week of May 6, 2002.**Week of May 13, 2002—Tentative**

Thursday, May 16, 2002

9:25 a.m.

Affirmation Session (Public Meeting) (If
needed)

9:30 a.m.

Meeting with World Association of Nuclear
Operators (WANO) (Public Meeting)This meeting will be webcast live at the
Web address—www.nrc.gov

2:00 p.m.

Discussion of Intragovernmental Issues
(Closed—Ex. 9)**Week of May 20, 2002—Tentative**There are no meetings scheduled for the
Week of May 20, 2002.* The schedule for Commission meetings is
subject to change on short notice. To verify
the status of meetings call (recording)—(301)
415-1292. Contact person for more
information: David Louis Gamberoni (301)
415-1651.**Additional Information**By a vote of 5-0 on April 4 and 5, the
Commission determined pursuant to
U.S.C. 552b(e) and § 9.107(a) of the
Commission's rules that "Discussion of
Security Issues (Closed—Ex. 1)" be held
on April 8, and on less than one week's
notice to the public.The NRC Commission Meeting
Schedule can be found on the Internet
at: [www.nrc.gov/what-we-do/policy-
making/schedule.html](http://www.nrc.gov/what-we-do/policy-making/schedule.html)This notice is distributed by mail to
several hundred subscribers; if you no
longer wish to receive it, or would like
to be added to the distribution, please
contact the Office of the Secretary,
Washington, DC 20555 (301-415-1969).
In addition, distribution of this meeting
notice over the Internet system is
available. If you are interested in
receiving this Commission meeting
schedule electronically, please send an
electronic message to dkw@nrc.gov.

Dated: April 11, 2002.

David Louis Gamberoni,*Technical Coordinator, Office of the
Secretary.*

[FR Doc. 02-9300 Filed 4-12-02; 12:26 pm]

BILLING CODE 7590-01-M**NUCLEAR REGULATORY
COMMISSION****Biweekly Notice Applications and
Amendments to Facility Operating
Licenses Involving No Significant
Hazards Considerations****I. Background**Pursuant to Public Law 97-415, the
U.S. Nuclear Regulatory Commission
(the Commission or NRC staff) is
publishing this regular biweekly notice.
Public Law 97-415 revised section 189
of the Atomic Energy Act of 1954, as
amended (the Act), to require the
Commission to publish notice of any
amendments issued, or proposed to be
issued, under a new provision of section
189 of the Act. This provision grants the
Commission the authority to issue and
make immediately effective any
amendment to an operating license
upon a determination by the
Commission that such amendment
involves no significant hazards
consideration, notwithstanding the
pendency before the Commission of a
request for a hearing from any person.This biweekly notice includes all
notices of amendments issued, or
proposed to be issued from March 22,
2002 through April 4, 2002. The last
biweekly notice was published on April
2, 2002 (67 FR 15619).**Notice of Consideration of Issuance of
Amendments to Facility Operating
Licenses, Proposed No Significant
Hazards Consideration Determination,
and Opportunity for a Hearing**The Commission has made a
proposed determination that the
following amendment requests involve
no significant hazards consideration.
Under the Commission's regulations in
10 CFR 50.92, this means that operation
of the facility in accordance with the
proposed amendment would not (1)
involve a significant increase in the
probability or consequences of an
accident previously evaluated; or (2)
create the possibility of a new or
different kind of accident from any
accident previously evaluated; or (3)
involve a significant reduction in a
margin of safety. The basis for this
proposed determination for each
amendment request is shown below.The Commission is seeking public
comments on this proposeddetermination. Any comments received
within 30 days after the date of
publication of this notice will be
considered in making any final
determination.Normally, the Commission will not
issue the amendment until the
expiration of the 30-day notice period.
However, should circumstances change
during the notice period such that
failure to act in a timely way would
result, for example, in derating or
shutdown of the facility, the
Commission may issue the license
amendment before the expiration of the
30-day notice period, provided that its
final determination is that the
amendment involves no significant
hazards consideration. The final
determination will consider all public
and State comments received before
action is taken. Should the Commission
take this action, it will publish in the
Federal Register a notice of issuance
and provide for opportunity for a
hearing after issuance. The Commission
expects that the need to take this action
will occur very infrequently.Written comments may be submitted
by mail to the Chief, Rules and
Directives Branch, Division of
Administrative Services, Office of
Administration, U.S. Nuclear Regulatory
Commission, Washington, DC 20555-
0001, and should cite the publication
date and page number of this **Federal
Register** notice. Written comments may
also be delivered to Room 6D22, Two
White Flint North, 11545 Rockville
Pike, Rockville, Maryland, from 7:30
a.m. to 4:15 p.m. Federal workdays.
Copies of written comments received
may be examined at the NRC's Public
Document Room (PDR), located at One
White Flint North, 11555 Rockville Pike
(first floor), Rockville, Maryland. The
filing of requests for a hearing and
petitions for leave to intervene is
discussed below.By May 16, 2002, the licensee may file
a request for a hearing with respect to
issuance of the amendment to the
subject facility operating license and
any person whose interest may be
affected by this proceeding and who
wishes to participate as a party in the
proceeding must file a written request
for a hearing and a petition for leave to
intervene. Requests for a hearing and a
petition for leave to intervene shall be
filed in accordance with the
Commission's "Rules of Practice for
Domestic Licensing Proceedings" in 10
CFR part 2. Interested persons should
consult a current copy of 10 CFR 2.714,
which is available at the NRC's PDR,
located at One White Flint North, 11555
Rockville Pike (first floor), Rockville,
Maryland. Publicly available records

will be accessible from the Agencywide Documents Access and Management Systems (ADAMS) Public Electronic Reading Room on the internet at the NRC Web site, <http://www.nrc.gov/reading-rm/doc-collections/cfr/>. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of a hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner

must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemaking and Adjudications Staff, or may be delivered to the Commission's PDR, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland, by the above date. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to the attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for a hearing will not be entertained absent a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for

amendment which is available for public inspection at the Commission's PDR, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible from the Agencywide Documents Access and Management Systems (ADAMS) Public Electronic Reading Room on the internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC PDR Reference staff at 1-800-397-4209, 304-415-4737 or by e-mail to pdr@nrc.gov.

Calvert Cliffs Nuclear Power Plant, Inc., Docket Nos. 50-317 and 50-318, Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2, Calvert County, Maryland

Date of amendments request: January 31, 2002.

Description of amendments request: The proposed amendments would change the method of verifying the boron concentration of each safety injection tank. Rather than taking a sample from each tank every 31 days, the proposed change would require leakage into the tanks to be monitored every 12 hours and a sample be taken every 6 months.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Would not involve a significant increase in the probability or consequences of an accident previously evaluated.

Boron concentration is controlled in the safety injection tanks (SITs) to prevent either excessive boron concentrations or insufficient boron concentrations. Post-loss-of-coolant accident (LOCA) emergency procedures directing the operator to establish simultaneous hot and cold leg injection are based on the worst case minimum boron precipitation time. Maintaining the maximum SIT boron concentration within the upper limit ensures that the SITs do not invalidate this calculation. The minimum boron requirements of 2300 ppm [parts per million] are based on beginning-of-life reactivity values and are selected to ensure that the reactor will remain subcritical during the reflood stage of a large break LOCA. During a large break LOCA, all control element assemblies are assumed not to insert into the core, and the initial reactor shutdown is accomplished by void formation during blowdown.

Sufficient boron concentration must be maintained in the SITs to prevent a return to criticality during reflood. Level and pressure instrumentation is provided to monitor the availability of the tanks during plant operation.

The Technical Specification Surveillance Requirement (SR 3.5.1.4) verifies that the boron concentration remains within the required range by sampling. Currently, the boron concentration in each SIT is required to be verified by taking a sample of the water in the SIT every 31 days. A containment entry is required to take a sample from each of the four SITs. In addition, the boron concentration of the water added to the SITs is also sampled at the discharge of the high pressure safety injection pump to ensure that the water being added to the SITs is within the required boron concentration limits prior to being added. All intentional sources of level increase have their boron concentrations administratively maintained to ensure that the SIT boron concentrations are within Technical Specification limits. However, the Reactor Coolant System boron concentration is lower during power operation than the boron concentration in the SITs. Two check valves in series prevent leakage from the Reactor Coolant System into the SITs.

This proposed amendment would require inleakage monitoring to be done every twelve hours in addition to taking samples from each SIT every six months. Samples would continue to be taken to verify the inleakage observations remain conservative. In addition, the requirement to sample the discharge of the operating high pressure safety injection pump prior to filling the SIT would remain.

As noted above, the SITs are used only to respond to an accident and are not an accident initiator. Therefore, the probability of an accident has not increased.

The engineering analysis and risk insights combine to demonstrate that the method of SIT boron concentration verification can be changed from sampling every 31 days to monitoring inleakage every twelve hours and sampling every six months. The inleakage monitoring is based on a calculation method that has sufficient conservatism to predict the boron concentration of the SITs as shown by sample. Therefore, the SITs would remain capable of responding to an accident as described above and the consequences of an accident previously evaluated are not increased.

Therefore the probability or consequences of an accident previously evaluated are not increased.

2. Would not create the possibility of a new or different [kind] of accident from any accident previously evaluated.

The proposed change does not alter the function of any equipment, nor has it to operate differently than it was designed to operate. All equipment required to mitigate the consequences of an accident would continue to operate as before. The proposed change alters the method of verification of the SIT boron concentration, but not the boron concentration requirements themselves.

Therefore, this change does not create the possibility of a new or different [kind] of accident from any accident previously evaluated.

3. Would not involve a significant reduction in a margin of safety.

The margin of safety defined by 10 CFR [Code of Federal Regulations] Part 100 has not been significantly reduced. The inleakage monitoring done to verify the concentration of boron in the SITs, is sufficiently conservative to ensure that the boron concentration would be underpredicted, leading to attempts to increase the boron concentration or a need to sample the affected SIT. Sampling of the SITs every six months will continue to be done to ensure that the inleakage monitoring remains conservative and representative. Water added to the SITs will also continue to be sampled to ensure that it meets the minimum boron concentrations. If the boron concentration is maintained in the SITs, the system operates as assumed in the Updated Final Safety Analysis Report Chapter 14 analyses and the analyses continue to meet the dose consequences acceptance criteria given in the Updated Final Safety Analysis Report.

Therefore, this proposed change does not involve a significant reduction in [a] margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendments request involves no significant hazards consideration.

Attorney for licensee: Jay E. Silberg, Esquire, Shaw, Pittman, Potts and Trowbridge, 2300 N Street, NW., Washington, DC 20037.

NRC Section Chief: Joel Munday, Acting.

Detroit Edison Company, Docket No. 50-341, Fermi 2, Monroe County, Michigan

Date of amendment request: February 21, 2002.

Description of amendment request: The proposed amendment involves

changes to the Fermi 2 Updated Final Safety Analysis Report (UFSAR) and Technical Requirements Manual which is incorporated by reference in the UFSAR to eliminate the chlorine detection function from the control room heating, ventilation, and air conditioning system. Changes to the UFSAR are subject to the requirements of 10 CFR 50.59; however, these changes are being submitted for Nuclear Regulatory Commission (NRC) review and approval since they involve the elimination of an automatic action in accordance with the Nuclear Energy Institute guidance document 96-07, Revision 1.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The chlorine detection system was originally added to the plant design when it was assumed that a chlorine rail car would be located on site for use in water treatment purposes; however, one-ton chlorine cylinders were used instead. In 1992, the use of chlorine for on site water treatment was discontinued. There is no chlorine stored on site and no significant amounts are stored at any other facility within the 5-mile radius of the plant. The only credible accident involving a chlorine release that could be carried into the control room is from a chlorine rail car accident on the three railroad tracks 3.4 to 3.8-miles away from the site. The probability of a rail car accident and spill of chlorine is not affected by the removal of the chlorine detectors located in the normal air intake for the CCHVAC [control room heating, ventilation and air conditioning] system; therefore, only the consequences of the event must be addressed as a result of the proposed change.

The chlorine detectors in the control room ventilation air intake are intended to provide protection to the control room occupants in the event of an accidental offsite chlorine release. Detroit Edison has performed a probabilistic risk assessment to determine the probability of reaching toxic chlorine concentration levels of 10 parts per million in the control room as a result of a chlorine railcar accident and spill within 5 miles of the plant. The probability analysis took no credit for any automatic or manual action to

isolate the control room. The results of the analysis show that the total probability of $8.4E-07$ per year is below the $1.0E-06$ threshold specified in Regulatory Guide (RG) 1.78, Revision 1. Therefore, since the probability analysis results meet the RG criteria, the elimination of the chlorine detection function will not significantly increase the consequences of an offsite chlorine release.

2. The change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The probabilistic risk assessment evaluation demonstrates that the likelihood of creating hazardous conditions in the control room as a result of a chlorine accident is very small. RG 1.78, Revision 1, states that events of such low frequencies do not need to be considered in the plant design because the resultant low levels of radiological risk are considered acceptable. The probabilistic assessment assumed no automatic or manual action to isolate the control room or to filter outside air before it is discharged in the control room. The evaluation did not rely on any structure, system or component to perform a specific function; therefore, the elimination of the chlorine detection system does not create the potential for a new or different kind of accident from any accident previously evaluated.

3. The change does not involve a significant reduction in the margin of safety.

The elimination of the chlorine detection system will not affect the protection of the control room operators from the hazard of an offsite chlorine release. No significant amounts of chlorine are stored within 5 miles of the plant and the only chlorine accident risk is from a railroad car accident over 3 miles away. The probabilistic evaluation demonstrates the low risk associated with a chlorine accident that would incapacitate the operators such that their functions in mitigating a radiological event are impacted. Since the Regulatory Positions in RG 1.78, Revision 1 are satisfied, deletion of the chlorine detection system will not result in a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: Peter Marquardt, Legal Department, 688 WCB,

Detroit Edison Company, 2000 2nd Avenue, Detroit, Michigan 48226-1279.

NRC Section Chief: William D. Reckley, Acting.

Dominion Nuclear Connecticut, Inc., Docket No. 50-336, Millstone Nuclear Power Station, Unit No. 2, New London County, Connecticut

Date of amendment request: February 5, 2002

Description of amendment request: The proposed amendment would revise the surveillance requirements associated with the Containment Isolation Valves (CIVs), Reactor Building Closed Cooling Water (RBCCW) System, and Service Water (SW) System. The proposed changes would remove redundant testing requirements that are already addressed by the Inservice Testing (IST) Program, which is required pursuant to Technical Specification 4.0.5, and would use Technical Specification 4.0.5 to control the specific acceptance criteria and frequency of test performance. Additional proposed changes would remove the post maintenance testing requirements associated with the CIVs, revise the wording of the RBCCW and SW Systems Limiting Conditions for Operation, and increase the allowed outage times for the RBCCW and SW Systems.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed Technical Specification changes associated with the limiting condition for operation requirements, surveillance requirements, and allowed outage times will not cause an accident to occur and will not result in any change in the operation of the associated accident mitigation equipment. The ability of the equipment associated with the proposed changes to mitigate the design basis accidents will not be affected. The proposed changes to the limiting condition for operation requirements will not affect the equipment operability requirements. The proposed surveillance requirements are adequate to ensure proper operation of the associated accident mitigation equipment. Proper operation of the containment isolation valves will still be verified, as appropriate, following maintenance activities. The proposed allowed outage times are reasonable and consistent with standard industry guidelines to ensure the accident

mitigation equipment will be restored in a timely manner. The design basis accidents will remain the same postulated events described in the Millstone Unit No. 2 Final Safety Analysis Report, and the consequences of those events will not be affected. Therefore, the proposed changes will not increase the probability or consequences of an accident previously evaluated.

The additional proposed changes to the Technical Specifications (e.g., combining requirements, deleting an expired footnote, and renumbering a requirement) will not result in any technical changes to the current requirements. Therefore, these additional proposed changes will not increase the probability or consequences of an accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes to the Technical Specifications do not impact any system or component that could cause an accident. The proposed changes will not alter the plant configuration (no new or different type of equipment will be installed) or require any unusual operator actions. The proposed changes will not alter the way any structure, system, or component functions, and will not alter the manner in which the plant is operated. There will be no effect on plant operation or accident mitigation equipment. The response of the plant and the operators following an accident will not be different. In addition, the proposed changes do not introduce any new failure modes. Therefore, the proposed changes will not create the possibility of a new or different kind of accident from any accident previously analyzed.

3. Involve a significant reduction in a margin of safety.

The proposed Technical Specification changes associated with the limiting condition for operation requirements, surveillance requirements, and allowed outage times will not cause an accident to occur and will not result in any change in the operation of the associated accident mitigation equipment. The equipment associated with the proposed Technical Specification changes will continue to be able to mitigate the design basis accidents as assumed in the safety analysis. The proposed surveillance requirements are adequate to ensure proper operation of the affected accident mitigation equipment. The proposed allowed outage times are reasonable and consistent with standard industry guidelines to ensure the accident

mitigation equipment will be restored in a timely manner. In addition, the proposed changes will not affect equipment design or operation, and there are no changes being made to the Technical Specification required safety limits or safety system settings. The proposed Technical Specification changes, in conjunction with existing administrative controls (e.g., IST Program), will provide adequate control measures to ensure the accident mitigation functions are maintained. Therefore, the proposed changes will not result in a reduction in a margin of safety.

The additional proposed administrative changes to the Technical Specifications (e.g., combining requirements, deleting an expired footnote, and renumbering a requirement) will not result in any technical changes to the current requirements. Therefore, these additional changes will not result in a reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: Lillian M. Cuoco, Senior Nuclear Counsel, Dominion Nuclear Connecticut, Inc., Rope Ferry Road, Waterford, CT 06385.
NRC Section Chief: James W. Clifford.

Dominion Nuclear Connecticut, Inc., Docket Nos. 50-336 and 50-423, Millstone Nuclear Power Station, Unit Nos. 2 and 3, New London County, Connecticut

Date of amendment request: February 14, 2002.

Description of amendment request: The proposed Technical Specification (TS) changes will relocate selected Millstone Units 2 and 3 TSs related to the Reactor Coolant System (RCS) and Plant Systems to the Technical Requirements Manual (TRM). The proposed TSs for Unit 2 include 3/4.4.9.1, "Pressure/Temperature Limits," 3/4.7.2, "Steam Generator Pressure/Temperature Limitation," 3/4.7.5, "Flood Level," 3/4.7.7, "Sealed Source Contamination," 3/4.7.8, "Snubbers," and related Tables, Figures, and Bases sections. The proposed TSs for Unit 3 include 3/4.4.9.1, "Pressure/Temperature Limits," 3/4.7.2, "Steam Generator Pressure/Temperature Limitation," 3/4.7.6, "Flood Protection," 3/4.7.10, "Snubbers," 3/4.7.11, "Sealed Source Contamination," 3/4.7.14, "Area Temperature

Monitoring," and corresponding Tables, Figures, and Bases sections.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed technical specification changes will relocate to the TRM the following items: surveillance requirements for the withdrawal of reactor vessel material irradiation specimens of Millstone Unit Nos. 2 and 3 which are part of the Pressure/Temperature Limits technical specifications, Millstone Unit Nos. 2 and 3 technical specifications covering Steam Generator Pressure/Temperature Limitation, Flood Level, Sealed Source Contamination, and Snubbers. Also the Millstone Unit No. 3 technical specification covering Area Temperature Monitoring will be relocated to the TRM. Since the relocated requirements remain the same, the proposed changes will have no effect on plant operation, or the availability or operation of any accident mitigation equipment. Therefore, the relocation of the requirements associated with these technical specifications will not impact an accident initiator and cannot cause an accident. These changes will not increase the probability or consequences of an accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed technical specification changes will relocate the requirements of selected Millstone Unit Nos. 2 and 3 technical specifications as described above to the TRM. The proposed changes do not alter the plant configuration (no new or different type of equipment will be installed) or require any new or unusual operator actions. Since the requirements remain the same, the proposed changes do not alter the way any system, structure, or component functions and do not alter the manner in which the plant is operated. The proposed changes do not introduce any new failure modes. Therefore, the proposed changes will not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Involve a significant reduction in a margin of safety.

The proposed technical specification changes will relocate to the TRM the following items: surveillance

requirements for the withdrawal of reactor vessel material irradiation specimens of Millstone Unit Nos. 2 and 3 which are part of the Pressure/Temperature Limits technical specifications, Millstone Unit Nos. 2 and 3 technical specifications covering Steam Generator Pressure/Temperature Limitation, Flood Level, Sealed Source Contamination, and Snubbers. Also the Millstone Unit No. 3 technical specification covering Area Temperature Monitoring will be relocated to the TRM. Since the proposed changes are solely to relocate the existing requirements, the proposed changes will have no effect on plant operation, or the availability or operation of any accident mitigation equipment. The plant response to the Design Basis Accidents will not change. Therefore, there will be no reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: Lillian M. Cuoco, Senior Nuclear Counsel, Dominion Nuclear Connecticut, Inc., Waterford, CT 06141-5127.

NRC Section Chief: James W. Clifford.

Nine Mile Point Nuclear Station, LLC, Docket No. 50-220, Nine Mile Point Nuclear Station Unit No. 1, Oswego County, New York

Date of amendment request: March 15, 2002.

Description of amendment request: The proposed amendment would revise the Nine Mile Point Unit 1 Technical Specifications (TSs), Table 4.6.4, "Shock Suppressors (Snubbers)," consistent with the model snubber visual inspection and acceptance requirements conveyed in Generic Letter 90-09, "Alternative Requirements for Snubber Visual Inspection and Corrective Actions."

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The operation of Nine Mile Point Unit 1 in accordance with the proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

Snubbers are utilized at Nine Mile Point Unit 1 (NMP1) to ensure the

structural integrity of the reactor coolant system and other safety-related (as well as certain non-safety related) systems during and following a seismic event or other event initiating dynamic loads. The proposed change to the snubber visual inspection schedule is based on that delineated in NRC [Nuclear Regulatory Commission] Generic Letter (GL) 90-09, "Alternative Requirements for Snubber Visual Inspection and Corrective Actions." This change does not modify any accident initiators or change any equipment or procedures used to limit the consequences of any accidents previously evaluated.

Accordingly, the proposed amendment will not significantly increase the probability or consequences of an accident previously evaluated.

2. The operation of Nine Mile Point Unit 1 in accordance with the proposed amendment will not create the possibility of a new or different kind of accident from any accident previously evaluated.

No physical modifications are being made to any snubbers or to any systems supported by snubbers by this proposed amendment. No method of plant or system operation is varied by use of the alternate snubber visual inspection schedule delineated in GL 90-09. Only the method utilized to determine future surveillance intervals for snubber visual inspections based on the previous inspection results is changed by the proposed amendment. This method was developed and published by the NRC in GL 90-09 for generic application at nuclear power plants.

Accordingly, the proposed amendment will not create the possibility of a new or different kind of accident from any accident previously evaluated.

The operation of Nine Mile Point Unit 1, in accordance with the proposed amendment, will not involve a significant reduction in a margin of safety.

In GL 90-09, the NRC staff determined that use of the alternate snubber visual inspection schedule by nuclear power plants will maintain the same level of confidence as the previous schedule required by the plants' Technical Specifications. GL 90-09 also recognized that snubber visual inspection is a complementary process to snubber functional testing and provides additional confidence in snubber operability. Snubber functional testing is not being modified by this proposed amendment.

Therefore, the proposed change will not adversely affect any structure, system, component, or function that is safety-related or important to safety.

Accordingly, the proposed amendment will not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: Mark J. Wetterhahn, Esquire, Winston & Strawn, 1400 L Street, NW., Washington, DC 20005-3502.

NRC Section Chief: Joel Munday, Acting.

Nuclear Management Company, LLC, Docket No. 50-305, Kewaunee Nuclear Power Plant, Kewaunee County, Wisconsin

Date of amendment request: March 19, 2002.

Description of amendment request: The proposed amendment would revise the Kewaunee Nuclear Power Plant accident source term used for design basis radiological analyses.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

Differences between the original source term and the proposed AST [accident source term] cannot affect the previously analyzed core damage frequency (CDF) and large early release frequency (LERF). Since there are no modifications proposed with this request for AST, Limiting Safety System Settings and Safety Limits specified in the Technical Specifications remain unchanged. Re-analysis of design basis accidents as described herein demonstrates that regulatory dose acceptance criteria continue to be satisfied. Thus, nothing in this proposal will cause an increase in the probability or consequence of an accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any accident previously evaluated.

There are no physical changes to the plant associated with this request, and the plant conditions for which [Nuclear Management Company] (NMC) evaluated design-basis accidents remain valid. Consequently, this proposal introduces no new failure modes. Thus, this proposal does not create the

possibility of a new or different kind of accident.

3. Involve a significant reduction in the margin of safety.

The revised design-basis accident offsite and control-room dose-calculations proposed herein remain within regulatory acceptance criteria set forth in 10 CFR 100 and 10 CFR 50 Appendix A, General Design Criterion 19. They also use the TEDE [total effective dose equivalent] dose acceptance criteria as directed by the Commission. An acceptable margin of safety is inherent in the limits described thereby. Thus, changes proposed by this request do not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: Bradley D. Jackson, Esq., Foley and Lardner, P.O. Box 1497, Madison, WI 53701-1497.

NRC Section Chief: William D. Reckley, Acting.

Nuclear Management Company, LLC, Docket Nos. 50-266 and 50-301, Point Beach Nuclear Plant, Units 1 and 2, Town of Two Creeks, Manitowoc County, Wisconsin

Date of amendment request: February 28, 2002.

Description of amendment request: The proposed amendment would revise technical specifications (TS) 1.1, "Definitions," "CREFS Actuation Instrumentation," TS 3.4.16, "RCS Specific Activity," TS 3.3.5, "CREFS Actuation Instrumentation," TS 3.4.16, "RCS Specific Activity," TS 3.7.9, "CREFS," and TS 3.7.13, "Secondary Specific Activity," and delete TS 3.9.3, "Containment Penetrations."

The accident source term used in the selection of the design-basis offsite and control room dose analysis would be replaced by the implementation of an alternative source term.

The specific TS changes would be as follows: (1) TS 1.1, "Definitions:" Revise the definition of L_a (containment leakage) by changing 0.4 percent to 0.2 percent. (2) TS 3.3.5, "CREFS Actuation Instrumentation:" Revise table 3.3.5-1 to indicate that either RE-101 or RE-235 must be operable to ensure that the control room radiation instrumentation necessary to initiate the CREFS emergency make-up mode is operable. Add the Control Room Area Monitor and Control Room Air Intake trip setpoints to Note "d" of table 3.3.5-1.

(3) TS 3.4.16, "RCS Specific Activity:" Revise LCO Action Condition A to indicate 1.0 $\mu\text{Ci/gm}$ as the maximum reactor coolant dose equivalent iodine 131 (DE I-131) value. Revise Figure 3.4.16-1 to indicate 60 $\mu\text{Ci/gm}$ DE I-131 as the maximum RCS limit for operations at or above 80 percent of rated thermal power. Revise SR 3.4.16.2 to verify 1.0 $\mu\text{Ci/gm}$ as the maximum reactor coolant DE I-131 value. (4) TS 3.7.9, "CREFS:" Delete SR 3.7.9.5. (5) TS 3.7.13, "Secondary Specific Activity:" Revise LCO 3.5.13 and SR 3.7.13 to indicate that the secondary specific activity shall be less than or equal to 0.1 $\mu\text{Ci/gm}$. (6) TS 3.9.3, "Containment Penetrations:" Delete Section 3.9.3.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration which is presented below:

1. Involve a significant increase in the probability or consequences of any accident previously evaluated.

The Alternative Source Term (AST) and those plant systems affected by implementing the proposed changes to the TS are not accident initiators and cannot increase the probability of an accident. The AST does not adversely affect the design or operation of the facility in a manner that would create an increase [in] the probability of an accident. Rather, the AST is used to evaluate the dose consequences of a postulated accident. The revised dose calculations, except those for LOCA, use the values in the proposed TS. The limiting design bases accidents at PBNP have been evaluated for implementation of the AST.

These analyses have demonstrated that, with the proposed changes, the dose consequences meet the regulatory acceptance criteria of 10 CFR 50.67 and RG 1.183. A comparison of the current offsite dose calculations to the revised offsite dose calculations indicate that the proposed changes will not result in a significant increase in the predicted dose consequences for any of the analyzed accidents. Therefore, the proposed changes do not involve a significant increase in the probability or consequences of any of the selected previously analyzed accidents.

2. Create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed amendment will not create the possibility for a new or different type of accident from any accident previously evaluated. Changes to the allowable activity in the primary and secondary systems do not result in

changes to the design or operation of these systems. The evaluation of the effects of the proposed changes indicates that all design standard and applicable safety criteria limits are met.

The systems affected by the changes are used to mitigate the consequences of an accident that has already occurred. The proposed TS changes and modifications do not significantly affect the mitigative function of these systems. Equipment important to safety will continue to operate as designed. Component integrity is not challenged. The changes do not result in any event previously deemed incredible being made credible. The changes do not result in more adverse conditions or result in any increase in the challenges to safety systems.

Therefore, the proposed changes do not create the possibility of a new or different type of accident from any accident previously evaluated.

3. Involve a significant reduction in a margin of safety.

The implementation of the proposed changes does not significantly reduce the margin of safety. These changes have been evaluated in the revisions to the analysis of the consequences of the design basis accidents for PBNP. The radiological analysis results in concert with the proposed TS changes, meet the regulatory acceptance criteria of 10 CFR 50.67 and RG 1.183. These acceptance criteria have been developed for the purpose of use in design basis accident analyses such that meeting these limits demonstrates adequate protection of public health and safety. The proposed changes will not degrade the plant protective boundaries, will not cause a release of fission products to the public and will not degrade the performance of any SSCs important to safety.

Therefore, the proposed changes to the TS would not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: John H. O'Neill, Jr., Shaw, Pittman, Potts, and Trowbridge, 2300 N Street, NW., Washington, DC 20037.

NRC Section Chief: William D. Reckley, Acting.

Nuclear Management Company, LLC, Docket Nos. 50-282 and 50-306, Prairie Island Nuclear Generating Plant, Units 1 and 2, Goodhue County, Minnesota

Date of amendment request: February 2, 2001, supplemented August 31, 2001.

Description of amendment request: The proposed amendments would revise the technical specifications (TSs) to clarify the plant conditions under which various specifications are applicable. The licensee stated in its amendment request that a literal reading of the current technical specifications wording may result in situations where a routine plant shutdown would seem to be prohibited by TSs and, thereby, require entry into TS 3.0.C. This amendment request also makes several administrative changes to the TSs, including revising references to the Chief Nuclear Corporate Officer, capitalizing defined terms, and updating references to previously relocated TS paragraphs and correcting the List of Figures. The licensee's supplement to the amendment request, dated August 31, 2001, proposed a correction of a typographical error in TS Table 3.5-2B, Action 33.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does operation of the facility with the proposed amendment involve a significant increase in the probability or consequences of an accident previously evaluated?

The proposed changes are administrative in nature and clarify existing specifications without reducing or altering the requirements imposed by existing specifications. The proposed changes do not significantly affect any system that is a contributor to initiating events for previously evaluated accidents. Neither do the changes significantly affect any system that is used to mitigate any previously evaluated accidents. Therefore, the proposed changes do not involve any significant increase in the probability or consequence of an accident previously evaluated.

2. Does operation of the facility with the proposed amendment create the possibility of a new or different kind of accident from any accident previously evaluated?

The proposed changes are administrative in nature and clarify existing specifications without reducing or altering the requirements imposed by existing specifications. The proposed

changes do not alter the design, function, or operation of any plant component and do not install any new or different equipment, therefore a possibility of a new or different kind of accident from those previously analyzed has not been created.

3. Does operation of the facility with the proposed amendment involve a significant reduction in a margin of safety?

The proposed changes are administrative in nature and clarify existing specifications without reducing or altering the requirements imposed by existing specifications. Thus, the proposed change[s] do not involve a significant reduction in the margin of safety associated with the safety limits inherent in either the princip[al] barriers to a radiation release (fuel cladding, RCS [reactor coolant system] boundary, and reactor containment), or the maintenance of critical safety functions (subcriticality, core cooling, ultimate heat sink, RCS inventory, RCS boundary integrity, and containment integrity).

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment requests involve no significant hazards consideration.

Attorney for licensee: Jay Silberg, Esq., Shaw, Pittman, Potts, and Trowbridge, 2300 N Street, NW., Washington, DC 20037.

NRC Section Chief: William D. Reckley, Acting.

Southern California Edison Company, et al., Docket Nos. 50-361 and 50-362, San Onofre Nuclear Generating Station, Units 2 and 3, San Diego County, California

Date of amendment requests: March 11, 2002.

Description of amendment requests: The proposed amendment would revise the Technical Specifications (TSs) for San Onofre Nuclear Generating Station, Units 2 and 3. Specifically, TS Section 1.1, Definitions, would be revised to change the definition of response time testing as it is applied to the Engineered Safety Features, and the Reactor Protective System. The proposed change is based on approved Technical Specification Task Force (TSTF) Traveler TSTF-368, Revision 0, "Incorporate Combustion Engineering Owners Group (CEOG) Topical Report to Eliminate Pressure Sensor Response Time Testing."

Basis for proposed no significant hazards consideration determination:

As required by 10 CFR 50.91(a), an analysis of the issue of no significant hazards consideration is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

The proposed amendment to the Technical Specification (TS) Definitions for Engineered Safety Feature (ESF) Response Time and Reactor Protective System (RPS) Response Time allows substitution of an allocated sensor response time in lieu of measuring sensor response time. Response time testing is not an initiator of any accident previously evaluated. Further, overall system response time will continue to meet Technical Specification requirements. The allocated sensor response times allowed in lieu of measurement have been determined to adequately represent the response time of the components such that the safety systems utilizing those components will continue to perform their accident mitigation function as assumed in the safety analysis. Therefore, the proposed change does not involve a significant increase in the probability or consequences of any accident previously evaluated.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No.

The proposed amendment to TS Section 1.1, "Definitions," allows the substitution of an allocated sensor response time in lieu of sensor response time testing for selected components. The proposed change does not involve a physical alteration of the plant (no new or different type of equipment will be installed) or a change in the methods governing normal plant operation. Therefore, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety?

Response: No.

The proposed amendment to TS Section 1.1, "Definitions," allows the substitution of an allocated sensor response time in lieu of measured sensor response time for certain pressure sensors. The allocated pressure sensor response times allowed in lieu of measurement have been determined to adequately represent the response time of the components such that the safety systems utilizing those components will continue to perform their accident

mitigation function as assumed in the safety analysis. Therefore, the proposed change does not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment requests involve no significant hazards consideration.

Attorney for licensee: Douglas K. Porter, Esquire, Southern California Edison Company, 2244 Walnut Grove Avenue, Rosemead, California 91770.

NRC Section Chief: Stephen Dembek.

Tennessee Valley Authority, Docket Nos. 50-327 and 50-328, Sequoyah Nuclear Plant, Units 1 and 2, Hamilton County, Tennessee

Date of application for amendments: March 4, 2002 (TS 00-04).

Brief description of amendments: The proposed amendment would change the Sequoyah (SQN) Unit 1 and 2 Technical Specification (TS) to relocate the current requirements for ice condenser ice bed temperature and inlet door position monitoring systems to the SQN Technical Requirements Manual (TRM). These relocated specifications are consistent with the latest version of the improved Standard TS (NUREG-1431). The affected functions have been evaluated in accordance with Title 10 of the *Code of Federal Regulations*, Section 50.36 (10 CFR 50.36) for applicability to the criteria for requirements that must be retained in the TS. In each case, the four criteria of 10 CFR 50.36 did not apply to these functions. This revision will provide better consistency between the SQN TS and NUREG-1431.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), Tennessee Valley Authority (TVA), the licensee, has provided its analysis of the issue of no significant hazards consideration, which is presented below:

A. The proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed revision relocates the ice bed temperature monitoring system and the inlet door position monitoring system to the TRM. Relocation to the TRM continues to provide an acceptable level of applicability to plant operation and requires revisions to be processed in accordance with the provisions in 10 CFR 50.59. Evaluations of revisions in accordance with 10 CFR 50.59 will continue to ensure that these specifications adequately control the

functions of ice bed temperature and inlet door positions to maintain safe operation of the plant. These systems are not postulated to be the initiator of a design basis accident. Since there are no changes to these functions and their operation will remain the same, the probability of an accident is not increased by relocating these requirements to the TRM. Additionally, the accident mitigation capability and offsite dose consequences associated with accidents will not change because these functions will not be altered by the proposed relocation. Therefore, the consequences of an accident are not increased by this relocation to the TRM and the control of revisions to these specifications in accordance with 10 FR 50.59.

B. The proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed revision will not alter the functions for the ice bed temperature or inlet door positions such that accident potential would be changed. The location of these specifications in the TRM and the performance of revisions in accordance with 10 CFR 50.59 will continue to maintain acceptable operability requirements. Therefore, the possibility of an accident of a new or different kind is not created by the proposed relocation and deletion.

C. The proposed amendment does not involve a significant reduction in a margin of safety.

The proposed specification relocation will not affect plant setpoints or functions that maintain the margin of safety. This is based on the relocation to the TRM. The TRM continues to maintain the same level of operability requirements and surveillance testing to adequately ensure functionality of the ice bed temperature monitoring system and the inlet door position monitoring system. The TRM is controlled in accordance with requirements of 10 CFR 50.59. Therefore, the proposed relocation and deletion is acceptable and will not reduce the margin of safety.

The NRC has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: General Counsel, Tennessee Valley Authority, 400 West Summit Hill Drive, ET 10H, Knoxville, Tennessee 37902.

NRC Section Chief: Richard P. Correia.

Tennessee Valley Authority, Docket Nos. 50-327 and 50-328, Sequoyah Nuclear Plant, Units 1 and 2, Hamilton County, Tennessee

Date of application for amendments: March 4, 2002 (TS 01-03)

Brief description of amendments: The proposed amendment would change the Sequoyah (SQN) Unit 1 and 2 Technical Specifications (TSs) to delete one definition and modify several subsections contained in TS Section 6.0, Administrative Controls. These proposed changes have been prepared based on existing NRC guidance. The changes are being proposed in the following areas:

- Definition 1.17—"Member(s) of the Public." (NUREG-1431, Revision 2)
- TS 6.2.2.g, Overtime. (TS Travelers Form (TSTF)-258, Revision 4)
- TS 6.3, Facility Staff Qualifications. (TSTF-258, Revision 4)
- TS 6.8.4.a.ii, Primary Coolant Sources Outside Containment. (TSTF-299)
- TS 6.8.4.f, Radioactive Effluent Controls Program. (TSTF-258, Revision 4 and TSTF-308, Revision 1)
- TS 6.8.4.i, Deletion of the "Configuration Risk Management Program." (10 CFR 50.65)
- The second paragraph in TS 6.9.1.5 associated with specific activity limits. (NUREG-1431, Revision 2)
- TS 6.9.1.14, Monthly Reactor Operating Report contents revision. (TSTF-258, Revision 4)
- TS 6.12, High Radiation Areas revision. (TSTF-258, Revision 4)
- TS 6.15, Deletion of Major Changes To Radioactive Waste Treatment Systems (Liquid, Gaseous, and Solid). (NUREG-1431, Revision 2)

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), Tennessee Valley Authority (TVA), the licensee, has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No. The proposed changes that involve the rewording or reformatting of the existing TSs do not involve technical changes. Therefore, this change is administrative and does not affect the initiators of analyzed events or assumed mitigation of accidents or transient events.

Three of the changes remove programs from TSs based on present regulatory controls. Specifically 10 CFR 50.59, 10 CFR 50.65, 10 CFR 50.71(e), 10

CFR 50.73, and Performance Indicator data. Based on the requirements residing in existing regulations it is acceptable to remove them from TS. Additionally, any changes to these programs will be evaluated based on regulatory requirements, no significant increase in the probability or consequences of an accident previously evaluated will be allowed.

Therefore, the proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No. The proposed changes do not involve a physical alteration of the plant (no new or different type of equipment will be installed) or changes in methods governing normal plant operation. Therefore, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety?

Response: No. The proposed changes will not reduce the margin of safety because they have no effect on any safety analysis assumptions. Additionally, the proposed programs to be removed from TSs are contained in existing plant programs required by existing regulations. Since any future changes to these programs will be evaluated, no significant reduction in a margin of safety will be allowed.

Therefore, the proposed change does not involve a significant reduction in a margin of safety.

Based on the above, TVA concludes that the proposed amendment(s) present no significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and, accordingly, a finding of "no significant hazards consideration" is justified.

The NRC has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: General Counsel, Tennessee Valley Authority, 400 West Summit Hill Drive, ET 10H, Knoxville, Tennessee 37902.

NRC Section Chief: Richard P. Correia.

Wolf Creek Nuclear Operating Corporation, Docket No. 50-482, Wolf Creek Generating Station, Coffey County, Kansas

Date of amendment request: February 21, 2002.

Description of amendment request: The proposed amendment would revise Required Actions for Limiting Conditions for Operation (LCOs) 3.3.1, "Reactor Trip System (RTS) Instrumentation;" 3.4.5, "RCS [Reactor Coolant System] Loops—MODE 3;" 3.4.6, "RCS Loops—MODE 4;" 3.4.7, "RCS Loops—MODE 5, Loops Filled;" 3.4.8, "RCS Loops—MODE 5, Loops Not Filled;" 3.8.2, "AC Sources—Shutdown;" 3.8.5, "DC Sources—Shutdown;" 3.8.8, "Inverters—Shutdown;" 3.8.10, "Distribution Systems—Shutdown;" 3.9.3, "Nuclear Instrumentation;" 3.9.5, "Residual Heat Removal (RHR) and Coolant Circulation—High Water Level;" and 3.9.6, "Residual Heat Removal (RHR) and Coolant Circulation—Low Water Level" in the Wolf Creek Generating Station Technical Specifications (TSs). The Required Actions proposed would suspend operations involving positive reactivity additions or RCS boron concentration reductions. In addition, the proposed amendment would revise Notes, for several of the above LCOs, that preclude reductions in RCS boron concentration. This amendment would revise these Required Actions and LCO Notes to allow small, controlled, safe insertions of positive reactivity, but limit the introduction of positive reactivity such that compliance with the required shutdown margin or refueling boron concentration limits will still be satisfied.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

Overall protection system performance will remain within the bounds of the previously performed accident analyses since there are no hardware changes. The RTS instrumentation and reactivity control systems will be unaffected. Protection systems will continue to function in a manner consistent with the plant design basis. All design, material, and construction standards that were applicable prior to the request are maintained.

The probability and consequences of accidents previously evaluated in the USAR [Updated Safety Analysis Report] are not adversely affected because the changes to the Required Actions and LCO Notes assure the limits on SDM [shutdown margin] and refueling boron concentration continue to be met, consistent with the analysis assumptions and initial conditions included within the safety analysis and licensing basis. The activities covered by this amendment application are routine operating evolutions. The proposed changes do not reduce the capability of reborating the RCS.

The equipment and processes used to implement RCS boration or dilution evolutions are unchanged and the equipment and processes are commonly used throughout the applicable MODES under consideration. There will be no degradation in the performance of, or an increase in the number of challenges imposed on, safety-related equipment assumed to function during an accident situation. There will be no change to normal plant operating parameters or accident mitigation performance.

The proposed changes will not alter any assumptions or change any mitigation actions in the radiological consequence evaluations in the USAR. Therefore, the proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

There are no hardware changes nor are there any changes in the method by which any safety-related plant system performs its safety function. This amendment will not affect the normal method of plant operation or change any operating limits. The proposed changes merely permit the conduct of normal operating evolutions when additional controls over core reactivity are imposed by the Technical Specifications. The proposed changes do not introduce any new equipment into the plant or alter the manner in which existing equipment will be operated. The changes to operating procedures are minor, with clarifications provided that required limits must continue to be met. No performance requirements or response time limits will be affected. These changes are consistent with assumptions made in the safety analysis and licensing basis regarding limits on SDM and refueling boron concentration.

No new accident scenarios, transient precursors, failure mechanisms, or limiting single failures are introduced as

a result of this amendment. There will be no adverse effect or challenges imposed on any safety-related system as a result of this amendment.

This amendment does not alter the design or performance of the 7300 Process Protection System, Nuclear Instrumentation System, or Solid State Protection System used in the plant protection systems.

Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

The proposed changes do not alter the limits on SDM or refueling boron concentration. The nominal trip setpoints specified in the Technical Specifications Bases and the safety analysis limits assumed in the transient and accident analyses are unchanged. None of the acceptance criteria for any accident analysis is changed.

There will be no effect on the manner in which safety limits or limiting safety system settings are determined nor will there be any effect on those plant systems necessary to assure the accomplishment of protection functions. There will be no impact on the overpower limit, departure from nucleate boiling ratio (DNBR) limits, heat flux hot channel factor (F_Q), nuclear enthalpy rise hot channel factor ($F\Delta H$), loss of coolant accident peak cladding temperature (LOCA PCT), peak local power density, or any other margin of safety. The radiological dose consequence acceptance criteria listed in the Standard Review Plan will continue to be met.

Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: Jay Silberg, Esq., Shaw, Pittman, Potts and Trowbridge, 2300 N Street, NW., Washington, DC 20037.

NRC Section Chief: Stephen Dembek.

Notice of Issuance of Amendments to Facility Operating Licenses

During the period since publication of the last biweekly notice, the Commission has issued the following amendments. The Commission has determined for each of these amendments that the application

complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for A Hearing in connection with these actions was published in the **Federal Register** as indicated.

Unless otherwise indicated, the Commission has determined that these amendments satisfy the criteria for categorical exclusion in accordance with 10 CFR 51.22. Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared for these amendments. If the Commission has prepared an environmental assessment under the special circumstances provision in 10 CFR 51.12(b) and has made a determination based on that assessment, it is so indicated.

For further details with respect to the action see (1) the applications for amendment, (2) the amendment, and (3) the Commission's related letter, Safety Evaluation and/or Environmental Assessment as indicated. All of these items are available for public inspection at the Commission's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible from the Agencywide Documents Access and Management Systems (ADAMS) Public Electronic Reading Room on the internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1-800-397-4209, 301-415-4737 or by e-mail to pdr@nrc.gov.

AmerGen Energy Company, LLC, et al., Docket No. 50-219, Oyster Creek Nuclear Generating Station, Ocean County, New Jersey

Date of application for amendment: June 7, 2001.

Brief description of amendment: The amendment revises the Oyster Creek Technical Specifications, Section 6.2.2.2.j, to allow either the Senior Manager-Operations or an Operations Manager to satisfy the Senior Reactor Operator-licensed requirement of this section.

Date of Issuance: March 25, 2002.

Effective date: March 25, 2002, and shall be implemented within 30 days of issuance

Amendment No.: 226.

Facility Operating License No. DPR-16: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: July 25, 2001 (66 FR 38757). The Commission's related evaluation of this amendment is contained in a Safety Evaluation dated March 25, 2002.

No significant hazards consideration comments received: No.

AmerGen Energy Company, LLC, et al., Docket No. 50-219, Oyster Creek Nuclear Generating Station, Ocean County, New Jersey

Date of application for amendment: April 6, 2001.

Brief description of amendment: The amendment allows the 24-month capacity test for the Diesel Generator Starting Batteries to be performed during plant shutdowns or during the 24-month on-line Diesel Generator inspection.

Date of Issuance: March 27, 2002.

Effective date: March 27, 2002 and shall be implemented within 30 days of issuance

Amendment No.: 227.

Facility Operating License No. DPR-16: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: June 12, 2001 (66 FR 31702). The Commission's related evaluation of this amendment is contained in a Safety Evaluation dated March 27, 2002.

No significant hazards consideration comments received: No.

Carolina Power & Light Company, Docket No. 50-325, Brunswick Steam Electric Plant, Unit 1, Brunswick County, North Carolina

Date of amendment request: September 18, 2001, as supplemented December 10, 2001, and March 5, 2002.

Description of amendment request: The amendment revises the Safety Limit Minimum Critical Power Ratio (SLMCPR) values contained in TS 2.1.1.2, and revises the SLMCPR values from 1.10 to 1.12 for two recirculation loop operation, and from 1.11 to 1.14 for single recirculation loop operation.

Date of issuance: March 22, 2002.

Effective date: March 22, 2002.

Amendment No.: 220.

Facility Operating License No. DPR-71: The amendment changes the Technical Specifications.

Date of initial notice in Federal Register: October 17, 2001 (66 FR 52797). The December 10, 2001, and

March 5, 2002, supplements contained clarifying information only, and did not change the initial no significant hazards consideration determination or expand the scope of the initial **Federal Register** notice.

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated March 22, 2002.

No significant hazards consideration comments received: No.

Duke Energy Corporation, Docket Nos. 50-269, 50-270, and 50-287, Oconee Nuclear Station, Units 1, 2, and 3, Oconee County, South Carolina

Date of application of amendments: March 29, 2001.

Brief description of amendments: The amendments remove the NOTE that temporarily waived the upper limits of Technical Specifications 3.8.1.9; thus, these amendments restore the original requirements of Surveillance Requirement 3.8.1.9. In addition, these amendments reduce the time delay specified in TS 3.8.1.17 from 12 seconds to 5 seconds. These amendments will be implemented when the digital governor modifications have been implemented on both Keowee Hydroelectric Units.

Date of Issuance: March 20, 2002.

Effective date: As of the date of completion of digital governor modifications on both Keowee Hydroelectric Units, and shall be implemented within 30 days of the date of completion of such modifications, but no later than April 30, 2005.

Amendment Nos.: 322/322/323.

Renewed Facility Operating License Nos. DPR-38, DPR-47, and DPR-55: Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: May 2, 2001 (66 FR 22029). The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated March 20, 2002.

No significant hazards consideration comments received: No.

Entergy Operations, Inc., Docket No. 50-368, Arkansas Nuclear One, Unit No. 2, Pope County, Arkansas

Date of application for amendment: October 2, 2001, as supplemented by letter dated January 31, 2002.

Brief description of amendment: The amendment changes the technical specifications definition of reactor trip system response time and engineered safety feature response time to allow use of either an allocated or a measured response time for select sensors in these two systems.

Date of issuance: March 26, 2002.

Effective date: As of the date of issuance to be implemented within 60 days from the date of issuance.

Amendment No.: 239.

Facility Operating License No. NPF-6: The amendment revised the Technical Specifications.

Date of initial notice in Federal Register: October 31, 2001 (66 FR 55016). The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 26, 2002.

No significant hazards consideration comments received: No.

Entergy Operations, Inc., Docket No. 50-368, Arkansas Nuclear One, Unit No. 2, Pope County, Arkansas

Date of application for amendment: October 2, 2001.

Brief description of amendment: The amendment relocates the Technical Specification requirement that the reactor core be subcritical for a minimum of 175 hours prior to discharge of more than 70 assemblies to the spent fuel pool, to the technical requirements manual.

Date of issuance: April 1, 2002.

Effective date: As of the date of issuance to be implemented within 30 days from the date of issuance.

Amendment No.: 240.

Facility Operating License No. NPF-6: The amendment revised the Technical Specifications.

Date of initial notice in Federal Register: October 31, 2001 (66 FR 55016). The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated April 1, 2002.

No significant hazards consideration comments received: No.

Entergy Nuclear Operations, Docket No. 50-247, Indian Point Nuclear Generating Unit No. 2, Westchester County, New York

Date of application for amendment: September 20, 2001.

Brief description of amendment: The amendment allows the one-time extension of the intervals for selected Technical Specification (TS) surveillance requirements associated with the volume control tank, residual heat removal system, emergency diesel generators, and shock suppressors (snubbers). In addition, the amendment: (1) Corrects the channel functional test interval in Items 3 and 4 of TS Table 4.10-2 and Items 4 and 5 of Table 4.10-4, (2) deletes the alternate inspection requirements for the steam generator snubbers, (3) removes the reference to a prior one-time extension of checks, calibrations, and tests for certain instrument channels in TS Table 4.1-1 that is no longer applicable.

The amendment would enable the tests to be performed during the next

refueling outage starting no later than November 19, 2002.

Date of issuance: March 27, 2002.

Effective date: As of the date of issuance to be implemented within 60 days.

Amendment No.: 225.

Facility Operating License No. DPR-26: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: October 31, 2001 (66 FR 55014). The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 27, 2002.

No significant hazards consideration comments received: No.

Entergy Nuclear Operations, Docket No. 50-247, Indian Point Nuclear Generating Unit No. 2, Westchester County, New York

Date of application for amendment: December 11, 2000, as supplemented on November 5 and December 7, 2001.

Brief description of amendment: The amendment revised Technical Specification (TS) 3.1.F.2.a, "Primary to Secondary Leakage," and TS 4.13.A.3.f, "Steam Generator Tube Inservice Surveillance," based on the prior replacement of the steam generators (SGs). Specifically, the changes (1) revise the primary to secondary leakage limits and (2) delete the requirements associated with tube sleeve repair, SG tube denting, and F* repair classification and criteria. The associated TS Bases have been modified accordingly. In addition, the amendment includes several related administrative changes.

Date of issuance: April 2, 2002.

Effective date: As of the date of issuance to be implemented within 31 days.

Amendment No.: 226.

Facility Operating License No. DPR-26: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: January 24, 2001 (66 FR 7673). The November 5 and December 7, 2001, letters provided clarifying information that did not expand the application beyond the scope of the notice or change the initial proposed no significant hazards consideration determination.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated April 2, 2002.

No significant hazards consideration comments received: No.

Entergy Operations, Inc., Docket No. 50-382, Waterford Steam Electric Station, Unit 3, St. Charles Parish, Louisiana

Date of amendment request: July 9, 2001, as supplemented by letters dated

October 23, 2001, January 17, and February 1, 2002.

Brief description of amendment: This Technical Specification (TS) change removes TS requirements that will no longer be applicable following replacement of the part-length control element assemblies with five-element full-length control element assemblies (CEAs) and removal of the four-element CEAs on the core periphery.

Date of issuance: March 21, 2002.

Effective date: As of the date of issuance and shall be implemented 60 days from the date of issuance.

Amendment No.: 182.

Facility Operating License No. NPF-38: The amendment revised the Technical Specifications.

Date of initial notice in Federal Register: August 8, 2001 (66 FR 41617). The supplement letters dated October 23, 2001, January 17, and February 1, 2002, contained clarifying information only, and did not change the initial no significant hazards consideration determination, or expand the scope of the initial application.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 21, 2002.

No significant hazards consideration comments received: No.

Entergy Operations, Inc., Docket No. 50-382, Waterford Steam Electric Station, Unit 3, St. Charles Parish, Louisiana

Date of amendment request: September 21, 2001, as supplemented by letters dated December 10, 2001, and January 16 and 21, 2002.

Brief description of amendment: This amendment authorizes changes to the Waterford Steam Electric Station, Unit 3, Operating License and Technical Specifications associated with an increase in the licensed power level from 3,390 Megawatts thermal (MWT) to 3,441 MWT. These changes are made possible by increased feedwater flow measurement accuracy to be achieved by utilizing high accuracy ultrasonic flow measurement instrumentation.

Date of issuance: March 29, 2002.

Effective date: As of the date of issuance and shall be implemented 60 days from the date of issuance.

Amendment No.: 183.

Facility Operating License No. NPF-38: The amendment revised the Operating License and Technical Specifications.

Date of initial notice in Federal Register: October 31, 2001 (66 FR 55017). The supplement letters dated December 10, 2001, and January 16 and 21, 2002, contained clarifying information only, and did not change the initial no significant hazards consideration determination, or expand the scope of the initial application.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 29, 2002.

No significant hazards consideration comments received: No.

Florida Power and Light Company, Docket Nos. 50-250 and 50-251, Turkey Point Plant, Units 3 and 4, Miami-Dade County, Florida

Date of application for amendments: January 16, 2002 as supplemented February 7, 2002.

Brief description of amendments: The amendments modified Technical Specification Surveillance Requirement 4.8.1.1.2.g.7 to permit performance of the required emergency diesel generator functional testing during power operation as an alternative to its performance during shutdown.

Date of issuance: March 21, 2002.

Effective date: As of the date of issuance and shall be implemented within 60 days of issuance.

Amendment Nos.: 221 and 215.

Facility Operating License Nos. DPR-31 and DPR-41: Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: February 5, 2002 (67 FR 5328). The licensee's February 7, 2002, supplemental information did not affect the original no significant hazards consideration determination, and did not expand the scope of the request as noticed on February 5, 2002.

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated March 21, 2002.

No significant hazards consideration comments received: No.

North Atlantic Energy Service Corporation, et al., Docket No. 50-443, Seabrook Station, Unit No. 1, Rockingham County, New Hampshire

Date of amendment request: August 6, 2001, as supplemented on November 2, 2001, and February 2, 2002.

Description of amendment request: The amendment changes the Technical Specifications Sections 1.9, "Core Alterations," 1.14, "Engineered Safety Features Response Time," and 1.29, "Reactor Trip Response Time."

Date of issuance: April 3, 2002.

Effective date: As of its date of issuance, and shall be implemented within 60 days.

Amendment No.: 81.

Facility Operating License No. NPF-86: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: November 28, 2001 (66 FR 59509). The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated April 3, 2002.

No significant hazards consideration comments received: No.

Nuclear Management Company, LLC, Docket Nos. 50-266 and 50-301, Point Beach Nuclear Plant, Units 1 and 2, Town of Two Creeks, Manitowoc County, Wisconsin

Date of application for amendments: November 1, 2001.

Brief description of amendments: These amendments revise the Technical Specifications to allow a one-time extension of the allowed outage time for the control room emergency filtration system (CREFS) from 7 days to 30 days. The licensee requested this one-time change in order to implement modifications to CREFS.

Date of issuance: March 29, 2002.

Effective date: As of the date of issuance and shall be implemented within 45 days.

Amendment Nos.: 203 and 208.

Facility Operating License Nos. DPR-24 and DPR-27: Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: November 28, 2001 (66 FR 59510). The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated March 29, 2002.

No significant hazards consideration comments received: No.

Omaha Public Power District, Docket No. 50-285, Fort Calhoun Station, Unit No. 1, Washington County, Nebraska

Date of amendment request: December 14, 2001.

Brief description of amendment: The amendment removes requirements for having the equipment hatch closed with four (4) bolts, and one door of the personnel access lock (PAL) closed during core alterations and refueling operations. The technical specifications (TS) for other containment penetrations were modified to be closed by an operable ventilation isolation actuation signal from one gaseous radiation monitor during core alterations and refueling operations. The amendment also modified the requirements for radiation monitors during core alterations and refueling operations. The TS Bases that were affected by the changes described above were modified. This amendment is based upon the alternate source term design basis site boundary and control room dose analyses previously reviewed and approved by the staff by Amendment No. 201 on December 14, 2001.

Date of issuance: March 26, 2002.

Effective date: March 26, 2002, and shall be implemented within 60 days from the date of its issuance. The

implementation of the amendment requires the commitments made by the licensee in Attachment 4 of its December 14, 2001, letter and as discussed in the staff's safety evaluation. These commitments are to be in place prior to any core alterations or refueling operations.

Amendment No.: 204.

Facility Operating License No. DPR-40: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: January 22, 2002 (67 FR 2926). The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 26, 2002.

No significant hazards consideration comments received: No.

Omaha Public Power District, Docket No. 50-285, Fort Calhoun Station, Unit No. 1, Washington County, Nebraska

Date of amendment request: December 14, 2001, as supplemented by letter dated March 21, 2002.

Brief description of amendment: The amendment revised Technical Specification 3.7(4) to allow the surveillance tests to be performed on a refueling frequency. In addition, the staff reviewed the documentation to correct the docket concerning inconsistencies in the 1973 Fort Calhoun Station (FCS) Safety Evaluation Report (SER) associated with the 13.8 kV transmission line capability associated with TS 3.7(4) in accordance with OPPD's request.

Date of issuance: March 26, 2002.

Effective date: March 26, 2002, to be implemented within 30 days from the date of issuance.

Amendment No.: 205.

Facility Operating License No. DPR-40: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: January 22, 2002 (67 FR 2927). The March 21, 2002, supplemental letter provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination. The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 26, 2002.

No significant hazards consideration comments received: No.

Southern California Edison Company, et al., Docket Nos. 50-361 and 50-362, San Onofre Nuclear Generating Station, Units 2 and 3, San Diego County, California

Date of application for amendments: March 21, 2001, as supplemented by letter dated January 11, 2002.

Brief description of amendments: The amendments revise the operating license of each unit to delete those license conditions that have been completed and are no longer required and to make other corrections and editorial changes.

Date of issuance: March 27, 2002.

Effective date: March 27, 2002, to be implemented within 30 days of issuance.

Amendment Nos.: Unit 2-185; Unit 3-176.

Facility Operating License Nos. NPF-10 and NPF-15: The amendments revised the Facility Operating Licenses.

Date of initial notice in Federal Register: April 18, 2001 (66 FR 20009). The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated March 27, 2002. The January 11, 2002, supplemental letter provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination.

No significant hazards consideration comments received: No.

Southern Nuclear Operating Company, Inc., Docket Nos. 50-348 and 50-364, Joseph M. Farley Nuclear Plant, Units 1 and 2, Houston County, Alabama

Date of amendments request: August 25, 2000, as supplemented by letter dated November 2, 2001.

Brief description of amendments: The amendments revise the Updated Final Safety Analysis Report described offsite dose analyses based on changes to the letdown flow rate and iodine spike postulated concurrent with the Main Steam Line Break or a Steam Generator Tube Rupture.

Date of issuance: April 4, 2002.

Effective date: As of the date of issuance and shall be implemented within 30 days from the date of issuance.

Amendment Nos.: 154/146.

Facility Operating License Nos. NPF-2 and NPF-8: Amendments revise the Updated Final Safety Analysis Report.

Date of initial notice in Federal Register: March 7, 2001 (66 FR 13807). The supplement dated November 2, 2001, provided clarifying information that did not change the scope of the August 5, 2000, application nor the initial proposed no significant hazards consideration determination.

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated April 4, 2002.

No significant hazards consideration comments received: No.

Tennessee Valley Authority, Docket Nos. 50-260 and 50-296, Browns Ferry Nuclear Plant, Units 2 and 3, Limestone County, Alabama

Date of application for amendments: August 17, 2001, as supplemented December 14, 2001, and February 6, 2002.

Brief description of amendments: The amendments revised the pressure-temperature limits for the reactor pressure vessel.

Date of issuance: March 28, 2002.

Effective date: As of date of issuance, to be implemented within 60 days.

Amendment Nos.: 275 and 233.

Facility Operating License Nos. DPR-52 and DPR-68: Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: September 19, 2001 (66 FR 48291). The December 14, 2001, and February 6, 2002, letters provided clarifying information that did not change the initial proposed no significant hazards consideration determination or expand the scope of the original **Federal Register** notice.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 28, 2002.

No significant hazards consideration comments received: No.

Tennessee Valley Authority, Docket No. 50-296, Browns Ferry Nuclear Plant, Unit 3, Limestone County, Alabama

Date of application for amendment: November 1, 2001, as supplemented March 15, 2002.

Brief description of amendment: The amendment revised the safety limit minimum critical power ratio values in Technical Specification 2.1.1.2.

Date of issuance: March 29, 2002.

Effective date: As of the date of issuance, to be implemented within 30 days.

Amendment No.: 234.

Facility Operating License No. DPR-68: Amendment revised the technical specifications.

Date of initial notice in Federal Register: January 8, 2002 (67 FRN 933). The March 15, 2002, letter provided clarifying information that did not change the scope of the original amendment request or the initial proposed no significant hazards consideration determination.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 29, 2002.

No significant hazards consideration comments received: No.

TXU Generation Company LP, Docket Nos. 50-445 and 50-446, Comanche Peak Steam Electric Station, Unit Nos. 1 and 2, Somervell County, Texas

Date of amendment request: June 19, 2002, as supplemented by letters dated August 15, August 31, November 20, and December 17, 2001.

Brief description of amendments: The application, as supplemented, requested that the antitrust conditions, contained in Appendix C of Facility Operating Licenses Nos. NPF-87 and NPF-89 for Comanche Peak Steam Electric Station, Units 1 and 2, respectively, be deleted.

Date of issuance: March 22, 2002.

Effective date: As of the date of issuance and shall be implemented within 30 days from the date of issuance.

Amendment Nos.: 94 and 94.

Facility Operating License Nos. NPF-87 and NPF-89: The amendments delete Appendix C from the Licenses.

Date of initial notice in Federal Register: August 20, 2001 (66 FR 43595). The supplemental letters provided clarifying information that did not change the staff's proposed no significant hazards consideration determination or expand the application beyond the scope of the **Federal Register** notice.

The Commission's related evaluation of the amendments are contained in a Safety Evaluation dated March 22, 2002, and its attachment.

No significant hazards consideration comments received: No.

TXU Generation Company LP, Docket Nos. 50-445 and 50-446, Comanche Peak Steam Electric Station, Unit Nos. 1 and 2, Somervell County, Texas

Date of amendment request: October 25, 2001, as supplemented by letter dated February 18, 2002.

Brief description of amendments: The amendments revise Technical Specification (TS) 4.2.1, "Fuel Assemblies," for Comanche Peak Steam Electric Station (CPSES), Units 1 and 2, to allow the use of ZIRLO™ test assemblies and to further allow, " * * * A limited number of lead test assemblies * * * be placed in non-limiting core regions."

Date of issuance: March 26, 2002.

Effective date: As of the date of issuance and shall be implemented within 60 days from the date of issuance.

Amendment Nos.: 95 and 95.

Facility Operating License Nos. NPF-87 and NPF-89: The amendments revised the Technical Specifications.

Date of initial notice in Federal Register: December 12, 2001 (66 FR

64306). The supplemental letter dated February 18, 2002, provided clarifying information that did not change the staff's original no significant hazards consideration determination or expand the scope of the original **Federal Register** notice.

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated March 26, 2002.

No significant hazards consideration comments received: No.

Wolf Creek Nuclear Operating Corporation, Docket No. 50-482, Wolf Creek Generating Station, Coffey County, Kansas

Date of amendment request: April 3, 2001, as supplemented by letters dated October 22 and December 18, 2001, and March 7, 2002.

Brief description of amendment: The amendment relocates certain reactor coolant system cycle-specific parameter limits from the technical specifications (TS) to the Core Operating Limits Report (COLR), and thus expands the COLR. Additionally, TS 5.6.5, "Core Operating Limits Report (COLR)," is revised to allow topical reports to be identified by title and number only.

Date of issuance: March 28, 2002.

Effective date: March 28, 2002, and shall be implemented, including relocating the requirements from the TSs to the COLR, as specified in the licensee's letters of April 3, October 22, and December 18, 2001, and March 7, 2002, and the Safety Evaluation attached to Amendment No. 144, prior to the startup from Refueling Outage 12, which is scheduled for the spring of 2002.

Amendment No.: 144.

Facility Operating License No. NPF-42: The amendment revised the Technical Specifications.

Date of initial notice in Federal Register: May 2, 2001 (66 FR 22036) and February 5, 2002 (67 FR 5342). The March 7, 2002, supplemental letter provided additional clarifying information that did not expand the scope of the application as noticed and did not change the staff's proposed no significant hazards consideration determination.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 28, 2002.

No significant hazards consideration comments received: No.

Notice of Issuance of Amendments to Facility Operating Licenses and Final Determination of No Significant Hazards Consideration and Opportunity for a Hearing (Exigent Public Announcement or Emergency Circumstances)

During the period since publication of the last biweekly notice, the Commission has issued the following amendments. The Commission has determined for each of these amendments that the application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Because of exigent or emergency circumstances associated with the date the amendment was needed, there was not time for the Commission to publish, for public comment before issuance, its usual 30-day Notice of Consideration of Issuance of Amendment, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing.

For exigent circumstances, the Commission has either issued a **Federal Register** notice providing opportunity for public comment or has used local media to provide notice to the public in the area surrounding a licensee's facility of the licensee's application and of the Commission's proposed determination of no significant hazards consideration. The Commission has provided a reasonable opportunity for the public to comment, using its best efforts to make available to the public means of communication for the public to respond quickly, and in the case of telephone comments, the comments have been recorded or transcribed as appropriate and the licensee has been informed of the public comments.

In circumstances where failure to act in a timely way would have resulted, for example, in derating or shutdown of a nuclear power plant or in prevention of either resumption of operation or of increase in power output up to the plant's licensed power level, the Commission may not have had an opportunity to provide for public comment on its no significant hazards consideration determination. In such case, the license amendment has been issued without opportunity for comment. If there has been some time for public comment but less than 30 days, the Commission may provide an

opportunity for public comment. If comments have been requested, it is so stated. In either event, the State has been consulted by telephone whenever possible.

Under its regulations, the Commission may issue and make an amendment immediately effective, notwithstanding the pendency before it of a request for a hearing from any person, in advance of the holding and completion of any required hearing, where it has determined that no significant hazards consideration is involved.

The Commission has applied the standards of 10 CFR 50.92 and has made a final determination that the amendment involves no significant hazards consideration. The basis for this determination is contained in the documents related to this action. Accordingly, the amendments have been issued and made effective as indicated.

Unless otherwise indicated, the Commission has determined that these amendments satisfy the criteria for categorical exclusion in accordance with 10 CFR 51.22. Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared for these amendments. If the Commission has prepared an environmental assessment under the special circumstances provision in 10 CFR 51.12(b) and has made a determination based on that assessment, it is so indicated.

For further details with respect to the action see (1) the application for amendment, (2) the amendment to Facility Operating License, and (3) the Commission's related letter, Safety Evaluation and/or Environmental Assessment, as indicated. All of these items are available for public inspection at the Commission's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible from the Agencywide Documents Access and Management Systems (ADAMS) Public Electronic Reading Room on the internet at the NRC Web site, <http://www.nrc.gov/NRC/ADAMS/index.html>. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document room (PDR) Reference staff at 1-800-397-4209, 304-415-4737 or by e-mail to pdr@nrc.gov.

The Commission is also offering an opportunity for a hearing with respect to the issuance of the amendment. By May 16, 2002, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose

interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland 20852, and electronically from the ADAMS Public Library component on the NRC Web site, <http://www.nrc.gov> (the Electronic Reading Room). If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of a hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of

the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses. Since the Commission has made a final determination that the amendment involves no significant hazards consideration, if a hearing is requested, it will not stay the effectiveness of the amendment. Any hearing held would take place while the amendment is in effect.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland 20852, by the above date. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-001, and to the attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for a hearing will not be entertained absent a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

Calvert Cliffs Nuclear Power Plant, Inc., Docket No. 50-318, Calvert Cliffs Nuclear Power Plant, Unit No. 2, Calvert County, Maryland

Date of application for amendment: April 1, 2002.

Brief description of amendment: The amendment increases the allowed outage time of one train of the control room emergency ventilation system from 14 to 21 days (for the loss of the emergency power supply only). This is a one-time change to support corrective maintenance and inspections of the 1A diesel generator during the Unit 1 refueling outage.

Date of issuance: April 4, 2002.

Effective date: As of the date of issuance.

Amendment No.: 227.

Renewed License No. DPR-69: Amendment revised the Technical Specifications.

Public comments requested as to proposed no significant hazards consideration: No. The Commission's related evaluation of the amendment, finding of emergency circumstances, and final determination of no significant hazards consideration, are contained in a Safety Evaluation dated April 4, 2002.

For the Nuclear Regulatory Commission.

Dated at Rockville, Maryland, this 8th day of April 2002.

Ledyard B. Marsh,

Acting Director, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 02-8866 Filed 4-15-02; 8:45 am]

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SECURITIES AND EXCHANGE COMMISSION

[Release No. 35-27515; 70-10019]

Filings Under the Public Utility Holding Company Act of 1935, as amended ("Act")

April 9, 2002.

Notice is hereby given that the following filing(s) has/have been made with the Commission pursuant to provisions of the Act and rules promulgated under the Act. All interested persons are referred to the application(s) and/or declaration(s) for complete statements of the proposed transaction(s) summarized below. The application(s) and/or declaration(s) and any amendment(s) is/are available for public inspection through the Commission's Branch of Public Reference.

Interested persons wishing to comment or request a hearing on the application(s) and/or declaration(s)