pension insurance modeling and research projects as well as electronic copies of documents created using electronic mail and work processing. Proposed for permanent retention are recordkeeping copies of such records as files pertaining to significant legislation, background papers and other records pertaining to selected regulations and policies, published and unpublished research reports and studies, and electronic databases consisting of longitudinal and transactional data, with the related system documentation.

15. Tennessee Valley Authority, Education, Training and Diversity (N1–142–00–1, 26 items, 24 temporary items). Records relating to employee and outreach training programs including such records as applications, lists of attendees, correspondence, forms, meeting notes, and brochures. Also included are electronic copies of documents created using electronic mail and word processing and an electronic information system containing data on training taken by agency employees. Proposed for permanent retention are recordkeeping copies of newsletters and files documenting the agency's overall strategic training needs.

Dated: February 11, 2002.

Michael J. Kurtz,

Assistant Archivist for Record Services—Washington, DC.

[FR Doc. 02–3943 Filed 2–15–02; 8:45 am]

BILLING CODE 7515–01–P

NUCLEAR REGULATORY COMMISSION

Agency Information Collection Activities: Proposed Collection; Comment Request

AGENCY: U.S. Nuclear Regulatory Commission (NRC).

ACTION: Notice of pending NRC action to submit an information collection request to OMB and solicitation of public comment.

SUMMARY: The NRC is preparing a submittal to OMB for review of continued approval of information collections under the provisions of the Paperwork Reduction Act of 1995 (44 U.S.C. chapter 35).

Information pertaining to the requirement to be submitted:

1. *The title of the information collection:* Exercise of Discretion for an Operating Facility, NRC Enforcement Policy (NUREG–1600)—(Notice of Enforcement Discretion (NOEDs)).

2. *Current OMB approval number:* 3150–0136.

3. *How often the collection is required:* On occasion.

4. *Who is required or asked to report:* Nuclear power reactor licensees and gaseous diffusion plant certificate holders.

5. *The number of annual respondents:* 17.

6. *The number of hours needed annually to complete the requirement or request:* 2,550.

7. *Abstract:* The NRC's Enforcement Policy addresses circumstances in which the NRC may exercise enforcement discretion. This enforcement discretion is designated as a Notice of Enforcement Discretion (NOED) and relates to circumstances which may arise where a nuclear power plant licensee's compliance with a Technical Specification Limiting Condition for Operation or with other license conditions would involve an unnecessary plant transient or performance of testing, inspection, or system realignment that is inappropriate for the specific plant conditions, or unnecessary delays in plant startup without a corresponding health and safety benefit. Similarly, for a gaseous diffusion plant, circumstances may arise where compliance with a Technical Safety Requirement or other condition would unnecessarily call for a total plant shutdown, or, notwithstanding that a safety, safeguards or security feature was degraded or inoperable, compliance would unnecessarily place the plant in a transient or condition where those features could be required. A licensee or certificate holder seeking the issuance of an NOED must provide a written justification, in accordance with guidance provided in NRC Inspection Manual, Part 9900, which documents the safety basis for the request and provides whatever other information the NRC staff deems necessary to decide whether or not to exercise discretion.

Submit, by April 22, 2002, comments that address the following questions:

1. Is the proposed collection of information necessary for the NRC to properly perform its functions? Does the information have practical utility?

2. Is the burden estimate accurate?

3. Is there a way to enhance the quality, utility, and clarity of the information to be collected?

4. How can the burden of the information collection be minimized, including the use of automated collection techniques or other forms of information technology?

A copy of the draft supporting statement may be viewed free of charge at the NRC Public Document Room, One White Flint North, 11555 Rockville Pike, Room O–1 F23, Rockville, MD 20852. OMB clearance requests are available at the NRC worldwide web site: *http://www.nrc.gov/NRC/PUBLIC/OMB/index.html.* The document will be available on the NRC home page site for 60 days after the signature date of this notice.

Comments and questions about the information collection requirements may be directed to the NRC Clearance Officer, Brenda Jo. Shelton, U.S. Nuclear Regulatory Commission, T–6 E6, Washington, DC 20555–0001, by telephone at 301–415–7233, or by Internet electronic mail at *INFOCOLLECTS@NRC.GOV.*

For the Nuclear Regulatory Commission.

Dated at Rockville, Maryland, this 12th day of February, 2002.

Brenda Jo. Shelton,

NRC Clearance Officer, Office of the Chief Information Officer.

[FR Doc. 02–3899 Filed 2–15–02; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–400]

Carolina Power & Light Company; Notice of Withdrawal of Application for Amendment to Facility Operating License

The U.S. Nuclear Regulatory Commission (the Commission) has granted the request of Carolina Power & Light Company (the licensee) to withdraw its February 15, 2001, application for proposed amendment to Facility Operating License No. NPF–63 for the Shearon Harris Nuclear Power Plant, Unit No. 1, located in Wake and Chatham Counties, North Carolina.

The proposed amendment would have revised the Technical Specifications to clarify and relocate the requirements for containment isolation valve testing.

The Commission had previously issued a Notice of Consideration of Issuance of Amendment published in the **Federal Register** on March 21, 2001 (66 FR 15917). However, by letter dated January 29, 2002, the licensee withdrew the proposed change.

For further details with respect to this action, see the application for amendment dated February 15, 2001, and the licensee's letter dated January 29, 2002, which withdrew the application for license amendment. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One

White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management Systems (ADAMS) Public Electronic Reading Room on the internet at the NRC Web site, *http://www.nrc.gov/reading-rm/adams/html.* Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS, should contact the NRC PDR Reference staff by telephone at 1–800–397–4209 or 301–415–4737, or by e-mail to *pdr@nrc.gov.*

Dated at Rockville, Maryland, this 11th day of February, 2002.

For the Nuclear Regulatory Commission.

John M. Goshen,

Project Manager, Section 2, Project Directorate II, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 02–3900 Filed 2–15–02; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–334]

Pennsylvania Power Company, Ohio Edison Company, FirstEnergy Nuclear Operating Company, Beaver Valley Power Station, Unit No. 1; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from the requirements of Title 10 of the Code of Federal Regulations (10 CFR), Section 50.60(a), and 10 CFR part 50, Appendix G, for Facility Operating License No. DPR–66, issued to FirstEnergy Nuclear Operating Company (the licensee), for operation of the Beaver Valley Power Station, Unit No. 1 (BVPS–1), located in Beaver County, Pennsylvania. Therefore, as required by 10 CFR 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

Appendix G to 10 CFR part 50 requires that pressure/temperature (P/T) limits be established for reactor pressure vessels during normal operating and hydrostatic or leak rate testing conditions. Specifically, this regulation states, ''The appropriate requirements on both the pressure-temperature limits and the minimum permissible temperature must be met for all conditions.'' Additionally, it specifies that the requirements for these limits are contained in the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code), Section XI, Appendix G.

To address provisions of an amendment to the Technical Specification P/T limits, the licensee requested in its application dated June 29, 2001, as supplemented by letters of October 4 and December 1, 2001, that the NRC staff exempt BVPS–1 from the requirements of 10 CFR, Section 50.60(a), and 10 CFR Part 50, Appendix G, to allow application of ASME Code Case N–640 in establishing the reactor vessel pressure limits at low temperatures.

ASME Code Case N–640 permits the use of an alternate reference fracture toughness (K_c fracture toughness curve instead of the K_a fracture toughness curve) for reactor vessel materials in determining the P/T limits. Since the K_c fracture toughness curve shown in ASME Code, Section XI, Appendix A, Figure A–2200–1 (the K_c fracture toughness curve), provides greater allowable fracture toughness than the corresponding K_a fracture toughness curve of ASME Code, Section XI, Appendix G, Figure G–2210–1 (the K_a fracture toughness curve), using Code Case N–640 for establishing the P/T limits would be less conservative than the methodology currently endorsed by 10 CFR part 50, Appendix G. Therefore, an exemption is required in order to apply ASME Code Case N–640.

The proposed action is in accordance with the licensee's application for exemption dated June 29, 2001, and supplements dated October 4 and December 1, 2001.

The Need for the Proposed Action

ASME Code Case N–640 is needed to revise the method used to determine the reactor coolant system (RCS) P/T limits.

The purpose of 10 CFR 50.60(a), and 10 CFR part 50, Appendix G, is to protect the integrity of the reactor coolant pressure boundary in nuclear power plants. This protection is accomplished through these regulations that, in part, specify fracture toughness requirements for ferritic materials of the reactor coolant pressure boundary. Pursuant to 10 CFR part 50, Appendix G, it is required that P/T limits for the RCS be at least as conservative as those obtained by applying the methodology of the ASME Code, Section XI, Appendix G.

Current overpressure protection system (OPPS) setpoints produce operational constraints by limiting the P/T range available to the operator to heat up or cool down the plant. The operating window through which the operator heats up and cools down the RCS becomes more restrictive with continued reactor vessel service. Reducing this operating window could potentially have an adverse safety impact by increasing the possibility of inadvertent OPPS actuation due to pressure surges associated with normal plant evolutions such as reactor coolant pump start and swapping operating charging pumps with the RCS in a water-solid condition. The impact on the P/T limits and OPPS setpoints has been evaluated for an increased service period to 22 effective full power years based on ASME Code, Section XI, Appendix G, requirements. The results indicate that the OPPS would significantly restrict the ability to perform plant heatup and cooldown, create an unnecessary burden to plant operations, and challenge control of plant evolutions required with OPPS enabled. Continued operation of BVPS–1 with P/T curves developed to satisfy ASME Code, Section XI, Appendix G, requirements without the relief provided by ASME Code Case N–640 would unnecessarily restrict the P/T operating window, especially at low-temperature conditions.

Application of ASME Code Case N–640 will provide results which are sufficiently conservative to ensure the integrity of the reactor coolant pressure boundary while providing P/T curves which are not overly restrictive.

In the associated exemption, the NRC staff would determine that, pursuant to 10 CFR 50.12(a)(2)(ii), the underlying purpose of the regulation will continue to be served by the implementation of ASME Code Case N–640.

Environmental Impacts of the Proposed Action

The NRC has completed its evaluation of the proposed action and concludes that there are no significant environmental impacts associated with the use of ASME Code Case N–640 to develop the new P–T limits and OPPS setpoints.

The proposed action will not significantly increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released off site, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does not involve any historic