

Signed at Washington, DC this 15 day of August, 2001.

Ann L. Combs,

Assistant Secretary, Pension and Welfare Benefits Administration.

[FR Doc. 01-21023 Filed 8-20-01; 8:45 am]

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DEPARTMENT OF LABOR

Pension and Welfare Benefits Administration

Working Group on Planning for Retirement Advisory Council on Employee Welfare and Pension Benefits Plans; Notice of Meeting

Pursuant to the authority contained in section 512 of the Employee Retirement Income Security Act of 1974 (ERISA), 29 U.S.C. 1142, a public meeting will be held Tuesday, September 11, 2001, of the Advisory Council on Employee Welfare and Pension Benefit Plans Working Group assigned to study planning for retirement.

The session will take place in Room N-5437 A-C, U.S. Department of Labor Building, Second and Constitution Avenue, NW., Washington, DC 20210. The purpose of the open meeting, which will run from 1 p.m. to approximately 4 p.m., is for working group members to conclude hearing testimony on ways in which individuals can be encouraged to better plan for retirement.

Members of the public are encouraged to file a written statement pertaining to the topic by submitting 20 copies on or before September 11, 2001, to Sharon Morrissey, Executive Secretary, ERISA Advisory Council, U.S. Department of Labor, Room N-5677, 200 Constitution Avenue, NW., Washington, DC 20210. Individuals or representatives of organizations wishing to address the Working Group should forward their request to the Executive Secretary or telephone (202) 219-8753. Oral presentations will be limited to 20 minutes, but an extended statement may be submitted for the record. Individuals with disabilities, who need special accommodations, should contact Sharon Morrissey by September 11, at the address indicated in this notice.

Organizations or individuals may also submit statements for the record without testifying. Twenty (20) copies of such statements should be sent to the Executive Secretary of the Advisory Council at the above address. Papers will be accepted and included in the record of the meeting if received on or before September 11.

Signed at Washington, DC this 15th day of August 2001.

Ann L. Combs,

Assistant Secretary, Pension and Welfare Benefits Administration.

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NATIONAL TRANSPORTATION SAFETY BOARD

Sunshine Act Meeting

TIME AND PLACE: 9:30 a.m., Tuesday, August 28, 2001.

PLACE: NTSB Conference Center, 429 L'Enfant Plaza, S.W., Washington, D.C. 20594.

STATUS: The one item is Open to the Public.

MATTER TO BE CONSIDERED:

7381 Highway Accident Report—Motorcoach Run-Off-the-Road Accident, New Orleans, Louisiana, on May 9, 1999.

NEWS MEDIA CONTACT: Telephone: (202) 314-6100. Individuals requesting specific accommodations should contact Ms. Carolyn Dargan at (202) 314-6305 by Friday, August 24, 2001.

FOR MORE FURTHER INFORMATION

CONTACT: Vicky D'Onofrio, (202) 314-6410.

Dated: August 17, 2001.

Vicky D'Onofrio,

Federal Register Liaison Officer.

[FR Doc. 01-21177 Filed 8-17-01; 1:56 pm]

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NUCLEAR REGULATORY COMMISSION

[Docket Nos. STN 50-454, STN 50-455, STN 50-456 AND STN 50-457]

Exelon Generation Company, LLC, Byron Station, Units 1 and 2, Braidwood Station, Units 1 and 2; Exemption

1.0 Background

The Exelon Generation Company, LLC, (the licensee) is the holder of Facility Operating License Nos. NPF-37, NPF-66, NPF-72 and NPF-77, which authorize operation of the Byron Station, Units 1 and 2, and Braidwood Station, Units 1 and 2. The licenses provide, among other things, that the facilities are subject to all rules, regulations, and orders of the U.S. Nuclear Regulatory Commission (NRC, the Commission) now or hereafter in effect.

Each of the above facilities consists of two pressurized water reactors. The

Byron units are located in Ogle County in Illinois and the Braidwood units are located in Will County in Illinois. This exemption refers to all four units.

2.0 Request/Action

Title 10 of the Code of Federal Regulations (10 CFR), part 50, appendix G, requires that pressure-temperature (P-T) limits be established for reactor pressure vessels (RPVs) during normal operating and hydrostatic or leak rate testing conditions. Specifically, 10 CFR part 50, appendix G states that "The appropriate requirements on both the pressure-temperature limits and minimum permissible temperature must be met for all conditions." Appendix G of 10 CFR part 50 specifies that the requirements for these limits are the American Society of Mechanical Engineers (ASME) Code, Section XI, Appendix G Limits.

To address provisions of amendments to the technical specifications (TS) P-T limits in the uprating submittal, the licensee requested in its supplement dated December 8, 2000, to its original submittal for power uprates for Byron and Braidwood, dated July 5, 2000, that the staff exempt Byron and Braidwood units from application of specific requirements of 10 CFR part 50, § 50.60(a) and appendix G, and substitute use of ASME Code Cases N-588 and N-640. This request, to apply these code cases to the proposed P-T limits, was later withdrawn by the licensee for application with the power uprate in a letter dated February 20, 2001. However, the licensee requested the NRC to complete its review of the exemption request for future P-T limit applications. Code Case N-588 permits the postulation of a circumferentially-oriented flaw (in lieu of an axially-oriented flaw) for the evaluation of the circumferential welds in RPV P-T limit curves, whereas, Code Case N-640 permits the use of alternate reference fracture toughness (K_{IC} fracture toughness curve instead of K_{Ia} fracture toughness curve) for reactor vessel materials in determining the P-T limits. Since the pressure stresses on a circumferentially-oriented flaw are lower than the pressure stresses on an axially-oriented flaw by a factor of 2, postulating a circumferentially-oriented flaw for the evaluation of the circumferential welds, as permitted by Code Case N-588, in establishing the P-T limits would be less conservative than the methodology currently endorsed by 10 CFR Part 50, Appendix G. Further, since the K_{IC} fracture toughness curve shown in ASME Section XI, Appendix A, Figure A-2200-1 provides greater allowable fracture toughness than the