

and Commission programs and projects on the Mississippi River and its tributaries; (2) District Commander's overview of current project issues within Memphis District; and (3) Views and comments on issues affecting programs or projects of the Commission and the Corps of Engineers.

TIME AND DATE: 8:30 a.m., August 21, 2001.

PLACE: On board MISSISSIPPI V at City Front, Greenville, MS.

STATUS: Open to the public.

MATTERS TO BE CONSIDERED: (1) Summary of national and regional issues affecting the Corps of Engineers and Commission programs and projects on the Mississippi River and its tributaries; (2) District Commander's overview of current project issues within Vicksburg District; and (3) Views and comments on issues affecting programs or projects of the Commission and the Corps of Engineers.

TIME AND DATE: 8:30 a.m., August 23, 2001.

PLACE: On board MISSISSIPPI V at City Front, Morgan City, LA.

STATUS: Open to the public.

MATTERS TO BE CONSIDERED: (1) Summary of national and regional issues affecting the Corps of Engineers and Commission and projects on the Mississippi River and its tributaries; (2) District Commander's overview of current project issues within New Orleans District; and (3) Views and comments on issues affecting programs or projects of the Commission and the Corps of Engineers.

CONTACT PERSON FOR MORE INFORMATION: Mr. Stephen Gambrell, telephone 601-634-5766.

Luz D. Ortiz,

Army Federal Register Liaison Officer.

[FR Doc. 01-18012 Filed 7-13-01; 4:22 pm]

BILLING CODE 3710-GX-M

NATIONAL FOUNDATION ON THE ARTS AND HUMANITIES

Cooperative Agreement for the Creation of a Toolbox for Individual Artists

AGENCY: National Endowment for the Arts.

ACTION: Notification of availability.

SUMMARY: The National Endowment for the Arts is requesting proposals leading to one (1) award of a Cooperative Agreement to compile a Toolbox is intended to assist individual artists in carrying out the business aspects of

their careers. Toolbox topics will include financial management, law, marketing, and insurance, and the Toolbox will contain sample forms, checklists, references, and procedures. Completion of the project is contemplated in one year. Those interested in receiving the Solicitation package should reference Program Solicitation PS 01-02 in their written request and include two (2) self-addressed labels. Verbal requests for the Solicitation will not be honored. It is anticipated that the Program Solicitation will also be posted on the Endowment's Web site at <http://www.arts.gov>.

DATES: Program Solicitation PS 01-02 is scheduled for release approximately August 6, 2001 with proposals due on September 4, 2001.

ADDRESSES: Requests for the Solicitation should be addressed to the National Endowment for the Arts, Grants & Contracts Office, Room 618, 1100 Pennsylvania Ave., NW, Washington, D.C. 20506.

FOR FURTHER INFORMATION CONTACT: William Hummel, Grants & Contracts Office, National Endowment for the Arts, Room 618, 1100 Pennsylvania Ave., NW, Washington, D.C. 20506 (202)/682-5482).

William I. Hummel,

Coordinator, Cooperative Agreements.

[FR Doc. 01-17885 Filed 7-17-01; 8:45 am]

BILLING CODE 7537-01-M

NUCLEAR REGULATORY COMMISSION

Atomic Safety and Licensing Board Panel

[Docket No. 50-213-OLA ASLBP No. 01-787-02-OLA]

Before Administrative Judges: Ann Marshall Young, Chair, Dr. Peter S. Lam, Thomas D. Murphy; In the Matter of Connecticut Yankee Atomic Power Company (Haddam Neck Plant); Notice of Hearing

July 12, 2001.

This proceeding concerns a license amendment application of Connecticut Yankee Atomic Power Company (CYAPCO), seeking to add a new license condition to approve a License Termination Plan (LTP) for its Haddam Neck Plant pursuant to 10 CFR 50.82(a)(9), (10), and allow CYAPCO to make changes to the approved LTP without prior NRC approval if certain criteria specified in the license condition are met. After a public meeting held October 17, 2000, NRC

Staff proposed to determine that the amendment request involves no significant hazards consideration under 10 CFR 50.92(c), and provided notice of this finding and of the opportunity for a hearing with regard to the amendment request. 65 FR 77913 (Dec. 13, 2000). In response to this notice, the Citizens Awareness Network (CAN) and the Connecticut Department of Public Utility Control (CDPUC) filed requests for hearing dated January 11 and 17, 2001, respectively. On January 31, 2001, an Atomic Safety and Licensing Board, consisting of the members listed above, was established to preside over this proceeding. 66 FR 9111 (Feb. 6, 2001).

Notice is hereby given that, by Memorandum and Order dated July 9, 2001, the Board granted Petitioners CAN and CDPUC a hearing, after holding oral argument in a series of prehearing conferences on the contentions and arguments of the Petitioners, CYAPCO and the NRC Staff. In this Memorandum and Order, the Board found that both CAN and CDPUC have standing to proceed, and admitted contentions relating to adequacy of the site characterization, scope of work required to meet the requirements of the rules, dose calculations, and water contamination.

This proceeding will be conducted under the Commission's hearing procedures set forth in 10 CFR part 2, subpart G. During the course of the proceeding, the Board may conduct additional oral argument as provided in 10 CFR 2.755, hold additional prehearing conferences pursuant to 10 CFR 2.752, and conduct an evidentiary hearing in accordance with 10 CFR 2.750-751. The time and place of these sessions will be announced in Licensing Board Orders. Except as limited by the parameters of telephone conferences (which will in any event be transcribed), members of the public are invited to attend any such sessions.

Additionally, as provided in 10 CFR 2.715(a), any person not a party to the proceeding may submit a written limited appearance statement setting forth his or her position on the issues in the proceeding. Persons wishing to submit a written limited appearance statement should send it to the Office of the Secretary, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Rulemakings and Adjudications Staff. A copy of the statement should also be served on the Chairman of the Atomic Safety and Licensing Board. At a later date, the Board will entertain oral limited appearance statements at a location or locations in the vicinity of the Haddam Neck Plant, which is located

approximately 21 miles southeast of Hartford, Connecticut, on the east bank of the Connecticut River. Notice of these oral limited appearance sessions will be published in the **Federal Register** and/or made available to the public at the NRC Public Document Room.

Documents related to this proceeding are available electronically through the Agencywide Documents access and Management System (ADAMS), with access to the public through the NRC's Internet Web site (Public Electronic Reading Room Link, <http://www.nrc.gov/NRC/ADAMS/index.html>). The NRC Public Documents Room (PDR) and many public libraries have terminals for public access to the Internet. Documents that may relate to this proceeding that are dated earlier than December 1, 1999, are available in microfiche form (with print form available on one-day recall) for public inspection at the PDR, Room 0-1 F21, NRC One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852-2738.

Dated: July 12, 2001.

For the Atomic Safety and Licensing Board.

Ann Marshall Young,

Chair, Administrative Judge.

[FR Doc. 01-17952 Filed 7-17-01; 8:45 am]

BILLING CODE 7590-01-M

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-354]

PSEG Nuclear, LLC, Hope Creek Generating Station; Exemption

1.0 Background

The PSEG Nuclear LLC (PSEG or the licensee) is the holder of Facility Operating License No. NPF-57 which authorizes operation of the Hope Creek Generating Station (HCGS). The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the U.S. Nuclear Regulatory Commission (NRC, the Commission) now or hereafter in effect.

The facility consists of a boiling water reactor located in Salem County in New Jersey.

2.0 Request/Action

Title 10 of the Code of Federal Regulations, part 50, appendix G, requires that pressure-temperature (P-T) limits be established for reactor pressure vessels (RPVs) during normal operating and hydrostatic or leak rate testing conditions. Specifically, 10 CFR part 50, appendix G, states that “[t]he

appropriate requirements on both the pressure-temperature limits and the minimum permissible temperature must be met for all conditions.” In addition, Appendix G of 10 CFR part 50 specifies that the requirements for these limits “must be at least as conservative as limits obtained by following the methods of analysis and the margins of safety of Appendix G of Section XI of the ASME Code [American Society of Mechanical Engineers Boiler and Pressure Vessel Code].”

By letter dated December 1, 2000, as supplemented by letters dated February 12, May 7, and May 14, 2001, PSEG submitted a license amendment request to increase the HCGS core thermal power level by 1.4 percent. The amendment request included proposed P-T limit curves for the HCGS RPV. As part of the same submittal, PSEG requested an exemption from specific requirements of 10 CFR 50.60(a) and Appendix G. The proposed exemption would allow the use of ASME Code Cases N-588, “Alternative to Reference Flaw Orientation of Appendix G for Circumferential Welds in Reactor Vessels, Section XI, Division 1,” and N-640, “Alternative Reference Fracture Toughness for Development of P-T Limit Curves for ASME Section XI, Division 1,” as alternative methods for complying with the fracture toughness requirements in 10 CFR Part 50, Appendix G. The proposed amendment relies, in part, on the requested exemption since the proposed P-T limit curves for the HCGS RPV were developed based on the use of Code Cases N-588 and N-640. Pursuant to 10 CFR 50.60(b), proposed alternatives to the requirements in Appendices G and H of 10 CFR Part 50 may be used by licensees when the Commission grants an exemption under 10 CFR 50.12.

3.0 Discussion

Pursuant to 10 CFR 50.12, the Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of 10 CFR part 50, when (1) the exemptions are authorized by law, will not present an undue risk to public health or safety, and are consistent with the common defense and security; and (2) when special circumstances are present. The licensee's application states that the proposed exemption meets the special circumstances provisions in 10 CFR 50.12(a)(2)(ii), which states that “[a]pplication of the regulation in the particular circumstances would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule.”

As previously discussed, the licensee has requested an exemption to use ASME Code Cases N-588 and N-640 as alternative methods for complying with the fracture toughness requirements in 10 CFR Part 50, Appendix G. The underlying purpose of 10 CFR part 50, appendix G, is to protect the integrity of the reactor coolant pressure boundary in nuclear power plants. This is accomplished through these regulations that, in part, specify fracture toughness requirements for ferritic materials of the reactor coolant pressure boundary. The staff's review related to each of the Code Cases is discussed below.

Code Case N-588

Code Case N-588 amends the provisions of the 1989 Edition of ASME Section XI, Appendix G, by permitting the postulation of a circumferentially oriented reference flaw as the limiting flaw in an RPV circumferential weld for the purpose of establishing RPV P-T limits. The 1989 Edition of ASME Section XI, Appendix G, would require that such a reference flaw be postulated as an axially oriented flaw in the circumferential weld.

The licensee addressed the technical justification for this exemption by citing industry experience and aspects of RPV fabrication which support the postulation of circumferentially oriented flaws for these welds. The reference flaw is a postulated flaw that accounts for the possibility of a prior existing defect that may have gone undetected during the fabrication process. Postulating the ASME Section XI, Appendix G reference flaw in a circumferential weld is physically unrealistic and overly conservative, because the length of the flaw is 1.5 times the vessel wall thickness, which is much longer than the width of the circumferential weld. Industry experience with the repair of weld indications found during preservice inspection, inservice nondestructive examinations, and data taken from destructive examination of actual vessel welds confirms that any remaining defects are small, laminar in nature, and do not cross transverse to the weld bead. Therefore, any postulated defects introduced during the fabrication process, and not detected during subsequent nondestructive examinations, would only be expected to be oriented in the direction of weld fabrication. ASME Code Case N-588 also provides appropriate procedures for determining the stress intensity factors for use in developing RPV P-T limits in accordance with ASME Code, Section XI, Appendix G, procedures. The procedures allowed by ASME Code Case