

NUCLEAR REGULATORY COMMISSION**[Docket No. 40-8681-MLA-9; ASLBP No. 01-789-01-MLA]****International Uranium (USA) Corporation; Designation of Presiding Officer**

Pursuant to delegation by the Commission, *see* 37 FR 28710 (Dec. 29, 1972), and the Commission's regulations, *see* 10 CFR 2.1201, 2.1207, notice is hereby given that (1) a single member of the Atomic Safety and Licensing Board Panel is designated as Presiding Officer to rule on petitions for leave to intervene and/or requests for hearing; and (2) upon making the requisite findings in accordance with 10 CFR 2.1205(h), the Presiding Officer will conduct an adjudicatory hearing in the following proceeding:

International Uranium (USA)

Corporation (Source Material License Amendment)

The hearing will be conducted pursuant to 10 CFR part 2, Subpart L, of the Commission's Regulations, "Informal Hearing Procedures for Adjudications in Materials and Operator Licensing Proceedings." This proceeding concerns a February 7, 2001 request for hearing submitted by the Glen Canyon Group of the Sierra Club. The request was filed in response to a request from International Uranium (USA) Corporation (IUSA) to amend its source material license to receive and process alternate feed materials at its Blanding, Utah White Mesa Uranium Mill from the Molycorp site located in Mountain Pass, California. The notice of receipt of the amendment and opportunity for a hearing was published in the **Federal Register** on January 9, 2001 (66 FR 1702).

The Presiding Officer in this proceeding is Administrative Judge Alan S. Rosenthal. Pursuant to the provisions of 10 CFR 2.722, 2.1209, Administrative Judge Richard F. Cole has been appointed to assist the Presiding Officer in taking evidence and in preparing a suitable record for review.

All correspondence, documents, and other materials shall be filed with Judges Rosenthal and Cole in accordance with 10 CFR 2.1203. Their addresses are:

Administrative Judge Alan S. Rosenthal,
Presiding Officer, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

Dr. Richard F. Cole, Special Assistant,
Atomic Safety and Licensing Board

Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

Issued at Rockville, Maryland, this 27th day of February 2001.

G. Paul Bollwerk III,

Chief Administrative Judge, Atomic Safety and Licensing Board Panel.

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NUCLEAR REGULATORY COMMISSION**[Docket No. 50-285]****Omaha Public Power District; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing**

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR-40 issued to Omaha Public Power District (the licensee) for operation of the Fort Calhoun Station, Unit No. 1, located in Washington County, Nebraska.

The proposed amendment would change the surveillance requirements for laboratory testing of the charcoal adsorbers for the control room, the spent fuel pool storage area and the safety injection pump rooms. In addition, the amendment would delete the laboratory testing requirements for the containment charcoal adsorbers. The changes comply with the guidance of Generic Letter (GL) 99-02, "Laboratory Testing of Nuclear-Grade Activated Charcoal."

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant

hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

Testing the control room, spent fuel pool storage area and safety injection pump rooms charcoal adsorbers in accordance with the requirements of ASTM D3803-1989 will not increase the probability or consequences of an accident previously evaluated. As noted in GL 99-02, testing to the new standards will strengthen the assurance the charcoal adsorbers will perform their design function during a Loss of Coolant Accident (LOCA). The ASTM D3803-1989 testing methodology is superior to the method OPPD [Omaha Public Power District] presently uses.

Removing credit for the containment charcoal adsorbers and replacing their function with the containment spray system will not involve a significant increase in the probability or consequences of an accident previously evaluated. This change is being accomplished in accordance with SRP [Standard Review Plan] 6.5.2. The containment spray system is an ESF [engineered safety feature] system and its operability is assured by Technical Specifications 2.4 and 3.6. In addition, the LOCA radiological consequences analyses were revised to re-confirm that OPPD is in compliance with SRP 6.4. The revised analyses resulted in a post-LOCA control room thyroid dose of 32 REM, which exceeds the SRP 6.4 limit of 30 REM. The SRP 6.4 dose limits are based on ICRP-2 dose methodology. The critical organ approach of ICRP-2 has been replaced by the ICRP-30 dose methodology that utilizes a weighted sum of doses to all irradiated organs and tissues. The applicable dose limits for analyses utilizing the ICRP-20 methodology are 5 REM for stochastic effects, 50 REM for all organs and tissues (e.g., thyroid), and 15 REM for the lens of the eye. The ICRP-30 dose methodology has been approved and implemented by the NRC through the new 10 CFR Part 20 regulation. Therefore, the calculated doses presented above are acceptable and meet the intent of SRP 6.4.

Finally, these changes will not affect non-credited functions of the containment charcoal adsorbers. The filters will be left in place, but not credited in the Loss of Coolant (LOCA) radiological consequences analyses. The filters will be tested in accordance with TS 3.6 (3) to verify they are not clogged by excessive amounts of foreign matter.

In conclusion, based on the discussion above, these changes will not significantly increase the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

Testing the control room, spent fuel pool storage area and safety injection pump rooms charcoal adsorbers in accordance with the requirements of ASTM D3803-1989 will not create the possibility of a new or different kind of accident from any accident previously evaluated. Testing to the new standards will strengthen the assurance the