

(GALL) Report, Standard Review Plan (SRP), and Regulatory Guide (Public Meeting) (Contact: Chris Grimes, 301-415-1183)

This meeting will be webcast live at the Web address—www.nrc.gov/live.html.

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SUPPLEMENTARY INFORMATION: By a vote of 5-0 on October 23, the Commission determined pursuant to U.S.C. 552b(e) and § 9.107(a) of the Commission's rules that "Affirmation of Final Rules—10 CFR Part 35, 'Medical Use of Byproduct Material' and 10 CFR Part 20, 'Standards for Protection Against Radiation'" be held on October 23, and on less than one week's notice to the public.

The NRC Commission Meeting Schedule can be found on the Internet at: <http://www.nrc.gov/SECY/smj/schedule.htm>

This notice is distributed by mail to several hundred subscribers; if you no longer wish to receive it, or would like to be added to it, please contact the Office of the Secretary, Attn: Operations Branch, Washington, D.C. 20555 (301-415-1661). In addition, distribution of this meeting notice over the Internet system is available. If you are interested in receiving this Commission meeting schedule electronically, please send an electronic message to wmh@nrc.gov or dkw@nrc.gov.

Dated: October 27, 2000.

William M. Hill, Jr.,

SECY Tracking Officer, Office of the Secretary.

[FR Doc. 00-28035 Filed 10-27-00; 2:15 pm]

BILLING CODE 7590-01-M

NUCLEAR REGULATORY COMMISSION

Notice of Availability for Referencing in License Amendment Applications—Model Safety Evaluation on Technical Specification Improvement To Eliminate Requirements on Post Accident Sampling Systems Using the Consolidated Line Item Improvement Process

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of availability.

SUMMARY: Notice is hereby given that the staff of the Nuclear Regulatory Commission (NRC) has prepared a model safety evaluation (SE) relating to the elimination of requirements on post

accident sampling imposed on licensees through orders, license conditions, or technical specifications. The NRC staff has also prepared a model no significant hazards consideration (NSHC) determination relating to this matter. The purpose of these models is to permit the NRC to efficiently process amendments that propose to remove requirements for the Post Accident Sampling System (PASS). Licensees of nuclear power reactors to which the models apply may request amendments, in accordance with Section 50.90 of Title 10 to the Code of Federal Regulations, confirming the applicability of the SE and NSHC determination to their reactors and providing the requested plant-specific verifications and commitments.

DATES: The period during which licensees may reference the model SE and NSHC determination expires October 31, 2001. Applications for amendments after this date must include plant-specific justifications for the proposed changes and an analysis about the issue of no significant hazards consideration.

FOR FURTHER INFORMATION CONTACT:

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SUPPLEMENTARY INFORMATION:

Background

Regulatory Issue Summary 2000-06, "Consolidated Line Item Improvement Process for Adopting Standard Technical Specification Changes for Power Reactors," was issued on March 20, 2000. The consolidated line item improvement process (CLIIP) is intended to improve the efficiency of NRC licensing processes. This is accomplished by processing proposed changes to the Standard Technical Specifications (STS) in a manner that supports subsequent license amendment applications. The CLIIP includes an opportunity for the public to comment on proposed changes to the STS following a preliminary assessment by the NRC staff and finding that the change will likely be offered for adoption by licensees. The CLIIP directs the NRC staff to evaluate any comments received for a proposed change to the STS and to either reconsider the change or to proceed with announcing the availability of the change for proposed adoption by licensees. Those licensees opting to apply for the subject change to their technical specifications are responsible for reviewing the staff's

evaluation, referencing the applicable technical justifications, and providing any necessary plant-specific information. Each amendment application made in response to the notice of availability would be processed and noticed in accordance with applicable rules and NRC procedures.

This proposed change was proposed for incorporation into the Standard Technical Specifications by the Westinghouse Owners Group (WOG) and the Combustion Engineering Owners Group (CEOG) participants in the Technical Specification Task Force (TSTF) and is designated TSTF-366. A notice of opportunity to comment on the use of CLIIP for the elimination of requirements for PASS and related administrative controls in technical specifications for plants with Westinghouse and Combustion Engineering designs was published in the **Federal Register** on August 11, 2000 (65 FR 49271). The nine comments submitted to the NRC staff in response to the solicitation are addressed later in this notice.

Applicability

This application of the CLIIP to remove requirements for PASS from technical specifications (and other elements of the licensing bases) is applicable to plants with Westinghouse and Combustion Engineering designs.

To efficiently process the incoming license amendment applications, the staff requests each licensee applying for the changes addressed by TSTF-366 using the CLIIP to address the plant-specific verifications and regulatory commitments that are identified in the model SE. The CLIIP does not prevent licensees from requesting an alternative approach or proposing the changes without the requested verifications and regulatory commitments. Licensees choosing to request an approach different than that described in this notice should submit applications with appropriate plant-specific justifications for the proposed changes and an analysis about the issue of no significant hazards consideration. Variations from the approach recommended in this notice may require additional review by the NRC staff and may increase the time and resources needed for the review.

In making the requested regulatory commitments, each licensee should address: (1) That the subject capability exists (or will be developed) and will be maintained; (2) where the capability or procedure will be described (e.g., severe accident management guidelines, emergency operating procedures, emergency plan implementing

procedures); and (3) a schedule for implementation. The amendment request need not provide details about designs or procedures. Each licensee should verify that it has, and make a regulatory commitment to maintain (or make a regulatory commitment to develop and maintain):

a. Contingency plans for obtaining and analyzing highly radioactive samples from the reactor coolant system, containment sump, and containment atmosphere;

b. A capability for classifying fuel damage events at the Alert level threshold (typically this is 300 $\mu\text{Ci/ml}$ dose equivalent iodine). This capability may use the normal sampling system and/or correlations of sampling or letdown line dose rates to coolant concentrations; and

c. The capability to monitor radioactive iodines that have been released to offsite environs.

Public Notices

The staff issued a **Federal Register** Notice (64 FR 66213, November 24, 1999) that requested public comment on the NRC's pending action to approve topical reports submitted by the WOG and the CEOG in which they proposed to eliminate regulatory requirements for PASS. In particular, the staff sought comment from offsite emergency response organizations so that any impact of the elimination of PASS on their response could be factored into the staff's evaluation. Appendices to the staff's safety evaluations for topical reports submitted by the CEOG and the WOG contain a synopsis of the public comments received and the staff's evaluation of the comments. The topical reports as well as the NRC staff's safety evaluations for the topical reports may be examined, and/or copied for a fee, at the NRC's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the ADAMS Public Library component on the NRC Web site, (the Electronic Reading Room). The staff's safety evaluations that address the public comments about the topical reports are available on ADAMS (Accession Numbers ML003715250 dated May 16, 2000, for the CEOG topical report and ML003723268 dated June 14, 2000, for the WOG topical report).

A notice soliciting comments from interested members of the public about the use of the CLIP for elimination of requirements for PASS was published in the **Federal Register** on August 11, 2000 (65 FR 49271). The staff received nine comments (six from individual

licensees, one from the Nuclear Energy Institute, one from a law firm that represents licensees, and one from a member of the public) as a result of the notice of opportunity to comment about the subject technical specification changes. Five of the letters received included general comments in favor of the CLIP and its use in eliminating requirements for PASS. Specific comments on the model SE were offered in four of the comment letters. The specific comments are discussed below:

1. A licensee suggested that the model SE include a discussion indicating that the contingency plans do not have to be carried out in emergency plans and exercises. A similar statement was included in the staff's SE for the topical report prepared by the WOG. The staff agrees with the comment and added a sentence to the model SE.

2. A licensee stated that some plants have safety-related hydrogen monitors with ranges significantly above hydrogen concentrations of 10% that could be used for severe accident conditions. The staff believes that the model SE provides the necessary flexibility for plant-specific differences in the ranges of the monitors by stating that the appropriate decision-makers may determine if a grab sample is necessary and practical during the management of a severe accident. A contingency plan for sampling the containment atmosphere also serves to confirm the indications from the monitors and provide information on parameters other than hydrogen concentrations (e.g., the mix of radionuclides) and should, for consideration of the amendment as part of the CLIP, be part of the plant-specific regulatory commitment discussed in the model SE. The staff did not revise the model SE in response to this comment.

3. A licensee suggested that the Alert level threshold (typically 300 $\mu\text{Ci/ml}$ dose equivalent iodine) recognize an alternative of 2% to 5% fuel clad damage and that instrumentation such as core exit thermocouples or radiation monitors might also be indicative of fuel clad damage. The staff did not intend to preclude the use of other parameters as an indication of the loss of or challenge to the fuel clad fission product barrier. The staff included the regulatory commitment (item 4.2) in the model SE to address classifying certain types of events (such as reactivity excursions or mechanical damage) which could cause fuel damage without having an indication of overheating on core exit thermocouples. The mention of normal sampling or letdown line dose rates in the model SE is intended to be alternatives for those licensees that

currently use PASS for assessing the 300 $\mu\text{Ci/ml}$ does equivalent iodine criterion for declaration of an Alert. The staff did not revise the model SE in response to this comment.

4. A commenter suggested that the use of the CLIP to eliminate PASS requirements be expanded to all licensed facilities. The staff may choose to use the CLIP to address the removal of PASS from plants with other than Westinghouse and Combustion Engineering designs. Such a use of the CLIP would follow a specific proposal and justification from the applicable owners groups similar to the TSTF submitted by the WOG and CEOG. The staff did not revise the model SE in response to this comment.

This notice is announcing the availability of the model safety evaluation and model NSHC determination for referencing in applications for amendments to technical specifications for applicable plants. Licensees wishing to adopt the change must submit an application in accordance with applicable regulatory requirements. The staff will in turn issue for each application a notice of consideration of issuance of amendment to facility operating license(s), a proposed NSHC determination, and an opportunity for a hearing. A notice of issuance of an amendment to operating license(s) will also be issued to announce the elimination of the PASS requirements for each plant that applies for and receives the requested change.

Model Safety Evaluation

U.S. Nuclear Regulatory Commission Office of Nuclear Reactor Regulation Consolidated Line Item Improvement Technical Specification Task Force (TSTF) Change TSTF-366

Elimination of Requirements for Post Accident Sampling System (PASS)

1.0 Introduction

In the aftermath of the accident at Three Mile Island (TMI), Unit 2, the Nuclear Regulatory Commission (NRC) imposed requirements on licensees for commercial nuclear power plants to install and maintain the capability to obtain and analyze post-accident samples of the reactor coolant and containment atmosphere. The desired capabilities of the Post Accident Sampling System (PASS) were described in NUREG-0737, "Clarification of TMI Action Plan Requirements." The NRC issued orders to licensees with plants operating at the time of the TMI accident to confirm the installation of PASS capabilities

(generally as they had been described in NUREG-0737). A requirement for PASS and related administrative controls was added to the technical specifications (TS) of the operating plants and was included in the initial TS for plants licensed during the 1980s and 90s. Additional expectations regarding PASS capabilities were included in Regulatory Guide 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants To Assess Plant and Environs Conditions During and Following an Accident."

Significant improvements have been achieved since the TMI accident in the areas of understanding risks associated with nuclear plant operations and developing better strategies for managing the response to potentially severe accidents at nuclear plants. Recent insights about plant risks and alternate severe accident assessment tools have led the NRC staff to conclude that some TMI Action Plan items can be revised without reducing the ability of licensees to respond to severe accidents. The NRC's efforts to oversee the risks associated with nuclear technology more effectively and to eliminate undue regulatory costs to licensees have prompted the NRC to consider eliminating the requirements for PASS in TS and other parts of the licensing bases of operating reactors.

The staff has completed its review of the topical reports submitted by the Combustion Engineering Owners Group (CEOG) and the Westinghouse Owners Group (WOG) that proposed the elimination of PASS. The justifications for the proposed elimination of PASS requirements center on evaluations of the various radiological and chemical sampling and their potential usefulness in responding to a severe reactor accident or making decisions regarding actions to protect the public from possible releases of radioactive materials. As explained in more detail in the staff's safety evaluations for the two topical reports, the staff has reviewed the available sources of information for use by decision-makers in developing protective action recommendations and assessing core damage. Based on this review, the staff found that the information provided by PASS is either unnecessary or is effectively provided by other indications of process parameters or measurement of radiation levels. The staff agrees, therefore, with the owners groups that licensees can remove the TS requirements for PASS, revise (as necessary) other elements of the licensing bases, and pursue possible design changes to alter or remove existing PASS equipment.

2.0 Background

In a letter dated May 5, 1999 (as supplemented by letter dated April 14, 2000), the CEOG submitted the topical report CE NPSD-1157, Revision 1, "Technical Justification for the Elimination of the Post-Accident Sampling System From the Plant Design and Licensing Bases for CEOG Utilities." A similar proposal was submitted on October 26, 1998 (as supplemented by letters dated April 28, 1999, April 10 and May 22, 2000), by the WOG in its topical report WCAP-14986, "Post Accident Sampling System Requirements: A Technical Basis." The reports provided evaluations of the information obtained from PASS samples to determine the contribution of the information to plant safety and accident recovery. The reports considered the progression and consequences of core damage accidents and assessed the accident progression with respect to plant abnormal and emergency operating procedures, severe accident management guidance, and emergency plans. The reports provided the owners groups' technical justifications for the elimination for the various PASS sampling requirements. The specific samples and the staff's findings are described in the following evaluation.

The NRC staff prepared this model safety evaluation (SE) relating to the elimination of requirements on post accident sampling and solicited public comment (65 FR 49271) in accordance with the consolidated line item improvement process (CLIIP). The use of the CLIIP in this matter is intended to help the NRC to efficiently process amendments that propose to remove the PASS requirements from TS. Licensees of nuclear power reactors to which this model apply were informed [FR] that they could request amendments confirming the applicability of the SE to their reactors and providing the requested plant-specific verifications and commitments.

3.0 Evaluation

The technical evaluations for the elimination of PASS sampling requirements are provided in the safety evaluations dated May 16, 2000, for the CEOG topical report CE NPSD-1157 and June 14, 2000, for the WOG topical report WCAP-14986. The NRC staff's safety evaluations approving the topical reports are located in the NRC's Agencywide Documents Access and Management System (ADAMS) (Accession Numbers ML003715250 for CE NPSD-1157 and ML003723268 for WCAP-14986).

The ways in which the requirements and recommendations for PASS were incorporated into the licensing bases of commercial nuclear power plants varied as a function of when plants were licensed. Plants that were operating at the time of the TMI accident are likely to have been the subject of confirmatory orders that imposed the PASS functions described in NUREG-0737 as obligations. The issuance of plant specific amendments to adopt this change, which would remove PASS and related administrative controls from TS, supersede the PASS specific requirements imposed by post-TMI confirmatory orders.

As described in its safety evaluations for the topical reports, the staff finds that the following PASS sampling requirements may be eliminated for plants of Combustion Engineering and Westinghouse designs:

1. Reactor coolant dissolved gases
2. Reactor coolant hydrogen
3. Reactor coolant oxygen
4. Reactor coolant pH
5. Reactor coolant chlorides
6. Reactor coolant boron
7. Reactor coolant conductivity
8. Reactor coolant radionuclides
9. Containment atmosphere hydrogen concentration
10. Containment oxygen
11. Containment atmosphere radionuclides
12. Containment sump pH
13. Containment sump chlorides
14. Containment sump boron
15. Containment sump radionuclides

The staff agrees that sampling of radionuclides is not required to support emergency response decision making during the initial phases of an accident because the information provided by PASS is either unnecessary or is effectively provided by other indications of process parameters or measurement of radiation levels. Therefore, it is not necessary to have dedicated equipment to obtain this sample in a prompt manner.

The staff does, however, believe that there could be significant benefits to having information about the radionuclides existing post-accident in order to address public concerns and plan for long-term recovery operations. As stated in the safety evaluations for the topical reports, the staff has found that licensees could satisfy this function by developing contingency plans to describe existing sampling capabilities and what actions (*e.g.*, assembling temporary shielding) may be necessary to obtain and analyze highly radioactive samples from the reactor coolant system (RCS), containment sump, and containment atmosphere. (See item 4.1

under Licensee Verifications and Commitments.) These contingency plans must be available to be used by a licensee during an accident; however, these contingency plans do not have to be carried out in emergency plan drills or exercises. The contingency plans for obtaining samples from the RCS, containment sump, and containment atmosphere may also enable a licensee to derive information on parameters such as hydrogen concentrations in containment and boron concentration and pH of water in the containment sump. The staff considers the sampling of the containment sump to be potentially useful in confirming calculations of pH and boron concentrations and confirming that potentially unaccounted for acid sources have been sufficiently neutralized. The use of the contingency plans for obtaining samples would depend on the plant conditions and the need for information by the decision-makers responsible for responding to the accident.

In addition, the staff considers radionuclide sampling information to be useful in classifying certain types of events (such as a reactivity excursion or mechanical damage) that could cause fuel damage without having an indication of overheating on core exit thermocouples. However, the staff agrees with the topical reports' contentions that other indicators of failed fuel, such as letdown radiation monitors (or normal sampling system), can be correlated to the degree of failed fuel. (See item 4.2 under Licensee Verifications and Commitments.)

In lieu of the information that would have been obtained from PASS, the staff believes that licensees should maintain or develop the capability to monitor radioactive iodines that have been released to offsite environs. Although this capability may not be needed to support the immediate protective action recommendations during an accident, the information would be useful for decision makers trying to limit the public's ingestion of radioactive materials. (See item 4.3 under Licensee Verifications and Commitments.)

The staff believes that the changes related to the elimination of PASS that are described in the topical reports, related safety evaluations and this proposed change to TS are unlikely to result in a decrease in the effectiveness of a licensee's emergency plan. Each licensee, however, must evaluate possible changes to its emergency plan in accordance with 10 CFR 50.54(q) to determine if the change decreases the effectiveness of its site-specific plan. Evaluations and reporting of changes to

emergency plans should be performed in accordance with applicable regulations and procedures.

The staff notes that redundant, safety-grade, containment hydrogen concentration monitors are required by 10 CFR 50.44(b)(1), are addressed in NUREG-0737 Item II.F.1 and Regulatory Guide 1.97, and are relied upon to meet the data reporting requirements of 10 CFR Part 50, Appendix E, Section VI.2.a.(i)(4). The staff concludes that during the early phases of an accident, the safety-grade hydrogen monitors provide an adequate capability for monitoring containment hydrogen concentration. The staff sees value in maintaining the capability to obtain grab samples for complementing the information from the hydrogen monitors in the long term (*i.e.*, by confirming the indications from the monitors and providing hydrogen measurements for concentrations outside the range of the monitors). As previously mentioned, the licensee's contingency plan (see item 4.1) for obtaining highly radioactive samples will include sampling of the containment atmosphere and may, if deemed necessary and practical by the appropriate decision-makers, be used to supplement the safety-related hydrogen monitors.

[Note 1—Each licensee should specify a desired implementation period for its specific amendment request. The implementation period would be that period necessary to develop and implement the items in 4.1 through 4.3 and, as necessary, to make other changes to documentation or equipment to support the elimination of PASS requirements. As an alternative, the licensee may choose to have a shorter implementation period and include the scheduling of items 4.1 through 4.3 as part of the regulatory commitments associated with this amendment request. Amendment requests that include commitments for implementation of the items in Section 4 within 6 months of the implementation of the revised TS will remain within the CLIP.]

[Note 2—There may be some collateral changes to the TS as a result of the removal of the administrative controls section for PASS. The following paragraphs address three potential changes that the staff is aware of (editorial changes, mention of PASS as a potential leakage source outside containment, and revision of the bases section for post accident monitoring instrumentation).

(A) The elimination of the TS and other regulatory requirements for PASS would result in additional changes to TS such as [*e.g.*, the renumbering of

sections or pages or the removal of references]. [If applicable: The elimination of PASS requirements requires the (elimination or modification) of Condition [2.C.x] in the operating license.] The changes are included in the licensee's application to revise the TS in order to take advantage of the CLIP. The staff has reviewed the changes and agrees that the revisions are necessary due to the removal of the TS section on PASS. The changes do not revise technical requirements beyond that reviewed by the NRC staff in connection with the supporting topical reports or the preparation of the TS improvement incorporated into the CLIP.

(B) The TS include an administrative requirement for a program to minimize to levels as low as practicable the leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident. The program includes preventive maintenance, periodic inspections, and leak tests for the identified systems. PASS is specifically listed in TS [5.5.2] as falling under the scope of this requirement. The applicability of this specification depends on whether or not PASS is maintained as a system that is a potential leakage path. (Note that several options (see following) exist for handling the impact that eliminating PASS requirements would have on the specification for the program to control leakage outside containment.)

(i) The licensee has stated that a plant change will be implemented such that PASS will not be a potential leakage path outside containment for highly radioactive fluids (*e.g.*, the PASS piping that penetrates the containment would be cut and capped). The modification will be made during the implementation period for this amendment such that it is appropriate to delete the reference to PASS in TS [5.5.2]. Requirements in NRC regulations (*e.g.*, 10 CFR Part 50, Appendix J) and other TS provide adequate regulatory controls over the licensee's proposed modification to eliminate PASS as a potential leakage path.

(ii) The licensee has stated that a plant change might be implemented such that PASS would not be a potential leakage path outside containment for highly radioactive fluids (*e.g.*, the PASS piping that penetrates the containment might be cut and capped). The modification will not, however, be made during the implementation period for this amendment. The licensee has proposed to add the following phrase to the reference to PASS in TS [5.5.2]: "(until such time as a modification

eliminates the PASS penetration as a potential leakage path.”

The above phrase makes clear that TS [5.5.2] remains applicable to the PASS as long as it is a possible leakage path and reflects that the actual modification of the piping system may be scheduled beyond the implementation period for this amendment. Requirements in NRC regulations (10 CFR Part 50, Appendix J) and other TS provide adequate regulatory controls over the licensee's modification to eliminate PASS as a potential leakage path. Following the modification to eliminate PASS as a potential leakage path, the licensee may elect (in order to maintain clarity and simplicity of the requirement) to revise TS [5.5.2] to remove the reference to PASS, including the phrase added by this amendment.

(iii) The licensee has stated that the configuration of the PASS will continue to be a potential leakage path outside containment for highly radioactive fluids (e.g., the PASS piping will penetrate the containment with valves or other components in the system from which highly radioactive fluid could leak). The licensee has [not proposed to change TS (5.5.2) or has changed TS (5.5.2) to revise the reference to this system from PASS to ()]. The staff agrees [that TS 5.5.2 is not affected or that the change to revise the reference from PASS to ()] is acceptable. A separate amendment request will be required if the licensee, subsequent to this amendment, decides to modify the plant to eliminate this potential leakage path and proposes to change the requirements of TS (5.5.2)].

(C) [Note-optional section if licensee provides markup of affected Bases pages] The elimination of PASS affects the discussion in the Bases section for TS [3.3.3, “Post Accident Monitoring Instrumentation”].

The current Bases mention the capabilities of PASS as part of the justification for allowing both hydrogen monitor channels to be out of service for a period of up to 72 hours. Although the licensee's application included possible wording for the revised Bases discussion for TS [3.3.3], the licensee will formally address the change to the Bases in accordance with [the Bases Control Program or its administrative procedure for revising Bases].

4.0 Verifications and Commitments

As requested by the staff in the notice of availability for this TS improvement, the licensee has addressed the following plant-specific verifications and commitments.

4.1 Each licensee should verify that it has, and make a regulatory

commitment to maintain (or make a regulatory commitment to develop and maintain), contingency plans for obtaining and analyzing highly radioactive samples of reactor coolant, containment sump, and containment atmosphere.

The licensee has [verified that it has or made a regulatory commitment to develop] contingency plans for obtaining and analyzing highly radioactive samples from the RCS, containment sump, and containment atmosphere. The licensee has committed to maintain the contingency plans within its [specified document or program]. The licensee has [implemented this commitment or will implement this commitment by (specified date)].

4.2 Each licensee should verify that it has, and make a regulatory commitment to maintain (or make a regulatory commitment to develop and maintain), a capability for classifying fuel damage events at the Alert level threshold (typically this is 300 $\mu\text{Ci/ml}$ dose equivalent iodine). This capability may utilize the normal sampling system and/or correlations of sampling or letdown line dose rates to coolant concentrations.

The licensee has [verified that it has or made a regulatory commitment to develop] a capability for classifying fuel damage events at the Alert level threshold. The licensee has committed to maintain the capability for the Alert classification within its [specified document or program]. The licensee has [implemented this commitment or will implement this commitment by (specified date)].

4.3 Each licensee should verify that it has, and make a regulatory commitment to maintain (or make a regulatory commitment to develop and maintain), the capability to monitor radioactive iodines that have been released to offsite environs.

The licensee has [verified that it has or made a regulatory commitment to develop] the capability to monitor radioactive iodines that have been released to offsite environs. The licensee has committed to maintain the capability for monitoring iodines within its [specified document or program]. The licensee has [implemented this commitment or will implement this commitment by (specified date)].

The NRC staff finds that reasonable controls for the implementation and for subsequent evaluation of proposed changes pertaining to the above regulatory commitments are provided by the licensee's administrative processes, including its commitment management program. Should the

licensee choose to incorporate a regulatory commitment into the emergency plan, final safety analysis report, or other document with established regulatory controls, the associated regulations would define the appropriate change-control and reporting requirements. The staff has determined that the commitments do not warrant the creation of regulatory requirements which would require prior NRC approval of subsequent changes. The NRC staff has agreed that NEI 99-04, Revision 0, “Guidelines for Managing NRC Commitment Changes,” provides reasonable guidance for the control of regulatory commitments made to the NRC staff. (See Regulatory Issue Summary 2000-17, Managing Regulatory Commitments Made by Power Reactor Licensees to the NRC Staff, dated September 21, 2000.) The commitments should be controlled in accordance with the industry guidance or comparable criteria employed by a specific licensee. The staff may choose to verify the implementation and maintenance of these commitments in a future inspection or audit.

5.0 State Consultation

In accordance with the Commission's regulations, the [] State official was notified of the proposed issuance of the amendments. The State official had [(1) no comments or (2) the following comments—with subsequent disposition by the staff].

6.0 Environmental Consideration

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (FR). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 Conclusion

The Commission has concluded, based on the considerations discussed above, that: (1) There is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Model No Significant Hazards Consideration Determination

Description of Amendment Request: The proposed amendment deletes requirements from the Technical Specifications (and, as applicable, other elements of the licensing bases) to maintain a Post Accident Sampling System (PASS). Licensees were generally required to implement PASS upgrades as described in NUREG-0737, "Clarification of TMI [Three Mile Island] Action Plan Requirements," and Regulatory Guide 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident." Implementation of these upgrades was an outcome of the lessons learned from the accident that occurred at TMI, Unit 2. Requirements related to PASS were imposed by Order for many facilities and were added to or included in the technical specifications (TS) for nuclear power reactors currently licensed to operate. Lessons learned and improvements implemented over the last 20 years have shown that the information obtained from PASS can be readily obtained through other means or is of little use in the assessment and mitigation of accident conditions.

The NRC staff issued a notice of opportunity for comment in the **Federal Register** on August 11, 2000 (65 FR 49271) on possible amendments to eliminate PASS, including a model safety evaluation and model no significant hazards consideration (NSHC) determination, using the consolidated line item improvement process. The NRC staff subsequently issued a notice of availability of the models for referencing in license amendment applications in the **Federal Register** on [] (65 FR). The licensee affirmed the applicability of the following NSHC determination in its application dated [].

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), an analysis of the issue of no significant

hazards consideration is presented below:

Criterion 1—The Proposed Change Does Not Involve a Significant Increase in the Probability or Consequences of an Accident Previously Evaluated

The PASS was originally designed to perform many sampling and analysis functions. These functions were designed and intended to be used in post accident situations and were put into place as a result of the TMI-2 accident. The specific intent of the PASS was to provide a system that has the capability to obtain and analyze samples of plant fluids containing potentially high levels of radioactivity, without exceeding plant personnel radiation exposure limits. Analytical results of these samples would be used largely for verification purposes in aiding the plant staff in assessing the extent of core damage and subsequent offsite radiological dose projections. The system was not intended to and does not serve a function for preventing accidents and its elimination would not affect the probability of accidents previously evaluated.

In the 20 years since the TMI-2 accident and the consequential promulgation of post accident sampling requirements, operating experience has demonstrated that a PASS provides little actual benefit to post accident mitigation. Past experience has indicated that there exists in-plant instrumentation and methodologies available in lieu of a PASS for collecting and assimilating information needed to assess core damage following an accident. Furthermore, the implementation of Severe Accident Management Guidance (SAMG) emphasizes accident management strategies based on in-plant instruments. These strategies provide guidance to the plant staff for mitigation and recovery from a severe accident. Based on current severe accident management strategies and guidelines, it is determined that the PASS provides little benefit to the plant staff in coping with an accident.

The regulatory requirements for the PASS can be eliminated without degrading the plant emergency response. The emergency response, in this sense, refers to the methodologies used in ascertaining the condition of the reactor core, mitigating the consequences of an accident, assessing and projecting offsite releases of radioactivity, and establishing protective action recommendations to be communicated to offsite authorities. The elimination of the PASS will not prevent an accident management strategy that meets the initial intent of

the post-TMI-2 accident guidance through the use of the SAMGs, the emergency plan (EP), the emergency operating procedures (EOP), and site survey monitoring that support modification of emergency plan protective action recommendations (PARs).

Therefore, the elimination of PASS requirements from Technical Specifications (TS) (and other elements of the licensing bases) does not involve a significant increase in the consequences of any accident previously evaluated.

Criterion 2—The Proposed Change Does Not Create the Possibility of a New or Different Kind of Accident From Any Previously Evaluated

The elimination of PASS related requirements will not result in any failure mode not previously analyzed. The PASS was intended to allow for verification of the extent of reactor core damage and also to provide an input to offsite dose projection calculations. The PASS is not considered an accident precursor, nor does its existence or elimination have any adverse impact on the pre-accident state of the reactor core or post accident confinement of radionuclides within the containment building.

Therefore, this change does not create the possibility of a new or different kind of accident from any previously evaluated.

Criterion 3—The Proposed Change Does Not Involve a Significant Reduction in the Margin of Safety

The elimination of the PASS, in light of existing plant equipment, instrumentation, procedures, and programs that provide effective mitigation of and recovery from reactor accidents, results in a neutral impact to the margin of safety. Methodologies that are not reliant on PASS are designed to provide rapid assessment of current reactor core conditions and the direction of degradation while effectively responding to the event in order to mitigate the consequences of the accident. The use of a PASS is redundant and does not provide quick recognition of core events or rapid response to events in progress. The intent of the requirements established as a result of the TMI-2 accident can be adequately met without reliance on a PASS.

Therefore, this change does not involve a significant reduction in the margin of safety.

Based upon the reasoning presented above and the previous discussion of the amendment request, the requested

change does not involve a significant hazards consideration.

Dated at Rockville, Maryland, this 25th day of October 2000.

For the Nuclear Regulatory Commission.

William D. Beckner,

*Chief, Technical Specification Branch,
Division of Regulatory Improvement
Programs, Office of Nuclear Reactor
Regulation.*

[FR Doc. 00-27941 Filed 10-30-00; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Draft Regulatory Guides; Issuance, Availability

The Nuclear Regulatory Commission has issued for public comment drafts of two new guides in its Regulatory Guide Series. This series has been developed to describe and make available to the public such information as methods acceptable to the NRC staff for implementing specific parts of the NRC's regulations, techniques used by the staff in evaluating specific problems or postulated accidents, and data needed by the staff in its review of applications for permits and licenses.

Draft Regulatory Guide DG-1102, "Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Post-Accident Engineered-Safety-Feature Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants," as a proposed Revision 3 to Regulatory Guide 1.52, is being developed to describe methods acceptable to the NRC staff for complying with the NRC's regulations with regard to the design, inspection, and testing criteria for air filtration and iodine adsorption units of engineered-safety-feature atmosphere cleanup systems in light-water-cooled nuclear power plants. This guide applies only to post-accident atmosphere cleanup systems that are designed to mitigate the consequences of postulated accidents.

Draft Regulatory Guide DG-1103, "Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Normal Ventilation Exhaust Systems in Light-Water-Cooled Nuclear Power Plants," as a proposed Revision 2 to Regulatory Guide 1.140, is being developed to present methods acceptable to the NRC staff for meeting the NRC's regulations with regard to the criteria for air filtration and adsorption units installed in the normal ventilation exhaust systems of light-water-cooled nuclear power plants.

These draft guides have not received complete staff approval and do not represent an official NRC staff position.

Comments may be accompanied by relevant information or supporting data. Written comments may be submitted to the Rules and Directives Branch, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Copies of comments received may be examined at the NRC Public Document Room, 11555 Rockville Pike, Rockville, MD. Comments will be most helpful if received by December 29, 2000.

You may also provide comments via the NRC's interactive rulemaking website through the NRC home page (<http://www.nrc.gov>). This site provides the availability to upload comments as files (any format), if your web browser supports that function. For information about the interactive rulemaking website, contact Ms. Carol Gallagher, (301) 415-5905; e-mail CAG@NRC.GOV. Electronic copies of these draft guides, under Accession Numbers ML003714744 for DG-8026 and ML003714764 for DG-8027, are available in NRC's Public Electronic Reading Room, which can also be accessed through NRC's web site, WWW.NRC.GOV. For information about the draft guides, contact Mr. J. Segala at (301) 415-1858; e-mail JPS1@NRC.GOV.

Although a time limit is given for comments on these draft guides, comments and suggestions in connection with items for inclusion in guides currently being developed or improvements in all published guides are encouraged at any time.

Regulatory guides are available for inspection at the Commission's Public Document Room, 11555 Rockville Pike, Rockville, MD. Requests for single copies of draft or final guides (which may be reproduced) or for placement on an automatic distribution list for single copies of future draft guides in specific divisions should be made in writing to the U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Reproduction and Distribution Services Section; or by fax to (301) 415-2289, or by email to DISTRIBUTION@NRC.GOV. Telephone requests cannot be accommodated. Regulatory guides are not copyrighted, and Commission approval is not required to reproduce them.

(5 U.S.C. 552(a)).

Dated at Rockville, Maryland, this 19th day of October 2000.

For the Nuclear Regulatory Commission.

Clare V. Kasputys,

*Deputy Director, Program Management,
Policy Development & Analysis Staff, Office
of Nuclear Regulatory Research.*

[FR Doc. 00-27939 Filed 10-30-00; 8:45 am]

BILLING CODE 7590-01-P

OFFICE OF MANAGEMENT AND BUDGET

Cost of Hospital and Medical Care Treatment Furnished by the United States; Certain Rates Regarding Recovery From Tortiously Liable Third Persons

By virtue of the authority vested in the President by Section 2(a) of Public Law 87-693 (76 Stat. 593; 42 U.S.C. 2652), and delegated to the Director of the Office of Management and Budget by Executive Order No. 11541 of July 1, 1970 (35 FR 10737), the three sets of rates outlined below are hereby established. These rates are for use in connection with the recovery, from tortiously liable third persons, of the cost of hospital and medical care and treatment furnished by the United States (Part 43, Chapter I, Title 28, Code of Federal Regulations) through three separate Federal agencies. The rates have been established in accordance with the requirements of OMB Circular A-25, requiring reimbursement of the full cost of all services provided. The rates are established as follows:

1. Department of Defense

The FY 2001 Department of Defense (DoD) reimbursement rates for inpatient, outpatient, and other services are provided in accordance with Title 10, United States Code, section 1095. Due to size, the sections containing the Drug Reimbursement Rates (section IV.C.) and the rates for Ancillary Services Requested by Outside Providers (section IV.D.) are not included in this package. Those rates are available from the TRICARE Management Activity's Uniform Business Office website, http://www.tricare.osd.mil/ebc/rm/rm_home.html. The medical and dental service rates in this package (including the rates for ancillary services and other procedures requested by outside providers) are effective October 1, 2000. Pharmacy rates are updated on an as needed basis.

2. Health and Human Services

The FY 2001 tortiously liable rates for Indian Health Service health facilities are based on Medicare cost reports. The obligations for the Indian Health Service hospitals participating in the cost report