

shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to Jeffrie J. Keenan, Esquire, Nuclear Business Unit—N21, P.O. Box 236,

Hancocks Bridge, NJ 08038, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)–(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated January 24, 2000, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and accessible electronically from the ADAMS Public Library component on the NRC Web site, <http://www.nrc.gov> (the Electronic Reading Room).

Dated at Rockville, Maryland, this 23rd day of February 2000.

For the Nuclear Regulatory Commission.

William C. Gleaves,

Project Manager, Section 2, Project Directorate I, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

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NUCLEAR REGULATORY COMMISSION

[Docket Nos.: 070-00784 and 040-07044]

Notice Consideration of the Approval of the Site Remediation Plan for the Formerly Licensed Union Carbide Facility in Lawrenceburg, TN

SUMMARY: The U. S. Nuclear Regulatory Commission (NRC) is considering a Remediation (Decommissioning) Plan (RDP) submitted by UCAR Carbon Company, Inc. (UCAR) to authorize decommissioning of its formerly licensed Union Carbide Corporation (UCC) facility in Lawrenceburg, Tennessee.

SUPPLEMENTARY INFORMATION: On August 19, 1998, UCAR submitted the RDP of its formerly licensed facility in Lawrenceburg, Tennessee. The RDP summarized the decommissioning activities that will be undertaken to remediate the contamination identified in three buildings, on an incinerator pad, and in the surrounding outdoor areas. Radioactive contamination at the UCC facility consists of building structures and soil contaminated with enriched uranium and thorium resulting

from licensed operation that occurred from 1963 to 1974.

NRC will require the licensee to remediate the UCC facility to meet NRC's decommissioning criteria, and during the decommissioning activities, to maintain effluents and doses within NRC requirements and as low as reasonably achievable.

Prior to approving the RDP, NRC will have made findings required by the Atomic Energy Act of 1954, as amended, and NRC's regulations. These findings will be documented in a Safety Evaluation Report and an Environmental Assessment.

UCAR has submitted the RDP and NRC hereby provides notice that the RDP is under review. Please address any questions or comments to the information contact person listed below.

The RDP for the formerly licensed UCC facility, Lawrenceburg, Tennessee, License Nos. SNM-00724 (Terminated) and SMB-00720 (Terminated), is available for inspection at the NRC's Public Document Room, 2120 L Street NW, Washington, DC 20555.

FOR FURTHER INFORMATION CONTACT:

Rebecca Tadesse, Decommissioning Branch, Division of Waste Management, Office of Nuclear Material Safety and Safeguards, at (301) 415-6221 or e-mail rxt@nrc.gov.

Dated at Rockville, Maryland, this 24th day of February 2000.

For the Nuclear Regulatory Commission.

Michael C. Layton,

Acting Chief, Decommissioning Branch, Division of Waste Management, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 00-4886 Filed 2-29-00; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Notice of the Staff's Intention to Combine Draft Regulatory Guide DG-4006 With the Standard Review Plan for Decommissioning

SUMMARY: The U. S. Nuclear Regulatory Commission (NRC) plans to combine the guidance in Draft Regulatory Guide DG-4006, with the Standard Review Plan (SRP) for decommissioning currently being developed by NRC staff.

SUPPLEMENTARY INFORMATION: In August 1998, NRC issued "Draft Regulatory Guide DG-4006, Demonstrating Compliance with the Radiological Criteria for License Termination" for a 2-year use and comment period. DG-4006 addressed the release from regulatory control of buildings and soil but did not pertain to the release of contaminated equipment. It included

regulatory positions on dose modeling, methods for conducting final status surveys, as low as reasonably achievable analysis, and license termination under restricted conditions. DG-4006 also discussed how these regulatory positions should be integrated during license termination activities. NRC staff initially intended to finalize the DG by July 2000. In September 1999, NRC staff stated that it would accept comments on DG-4006 until November 1999. NRC staff received approximately 185 comments on DG-4006 from four professional organizations, one Federal agency, three State regulatory agencies and the Conference of Radiation Control Program Directors, and two private concerns.

In late 1999, NRC staff, in recognition that similar guidance was being presented in the SRP, decided to combine the guidance in DG-4006 with the guidance in the SRP and use the SRP as the primary guidance document. This action will aid in consolidating decommissioning guidance in a user-friendly manner. As such, NRC staff does not plan to publish a final version of the Regulatory Guide. Comments submitted by interested individuals on DG-4006 will be considered as NRC staff finalizes the SRP.

FOR FURTHER INFORMATION CONTACT:
Dominick A. Orlando, Decommissioning Branch, Division of Waste Management, Office of Nuclear Material Safety and Safeguards, at (301) 415-6749.

Dated at Rockville, Maryland, this 24th day of February 2000.

For the Nuclear Regulatory Commission.

Michael C. Layton,

*Acting Chief, Decommissioning Branch,
Division of Waste Management, Office of
Nuclear Material Safety and Safeguards.*

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NUCLEAR REGULATORY COMMISSION

**[Docket Nos. STN 50-528, STN 50-529, and
STN 50-530]**

Arizona Public Service Company; Palo Verde Nuclear Generating Station, Units 1, 2, and 3, Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an amendment to Facility Operating Licenses Nos. NPF-41, NPF-51, and NPF-74, issued to the Arizona Public Service Company (the licensee), for operation of the Palo Verde Nuclear Generating Station (Palo Verde), Units 1,

2, and 3, located in Maricopa County, Arizona.

Environmental Assessment

Identification of Proposed Action

The proposed action would increase the number of fuel assemblies that can be stored in the Palo Verde spent fuel pools (SFPs) from 1034 fuel assemblies per SFP (1033 fuel assemblies for the Unit 2 SFP) to 1205 fuel assemblies per SFP. The increase in storage capacity is based on taking credit for fuel assembly burnup, for soluble boron, and for fuel assembly configuration in the SFP. In addition, the proposed action would increase the maximum radially averaged fuel enrichment from 4.3 weight percent to 4.8 weight percent.

The proposed action is in accordance with the licensee's application for amendments dated June 8, 1999, as supplemented by letters dated July 20 and November 24, 1999.

The Need for the Proposed Action

The licensee is planning on implementing dry cask storage in the second half of 2002. Since all three Palo Verde SFPs will lose the capacity to fully offload the core prior to that time, the licensee needs to increase the maximum number of fuel assemblies that can be stored in the SFPs. The higher enrichment limit is needed to provide flexibility in future core designs.

Environmental Impacts of the Proposed Action

Thermal Impact

The change in temperature of the SFP water was evaluated for the potential increase in reactivity. The current design basis for the SFP cooling system is based on the proposed increased capacity of the SFP, so no significant increase in SFP temperature is expected. In addition, because the reactivity coefficient in the SFP is negative, a temperature increase will result in a decrease in reactivity. Since increasing the capacity of the SFPs would increase the maximum heat load, the pool temperature would tend to be higher, not lower, after the proposed action was implemented. Therefore, the thermal impact of the proposed action would tend to increase the ability of the SFP system to maintain criticality parameters within the design bases of the plants.

The increased heat loads that result from increasing the SFP capacity would cause the total heat load rejected to the environment to increase. The maximum increase in heat rejection to the environment is less than 0.1 percent of

the total heat load rejected to the environment by an operating Palo Verde unit, and is not considered a significant impact to the environment.

Radioactive Waste Treatment

The Palo Verde units use waste treatment systems designed to collect and process gaseous, liquid, and solid waste that might contain radioactive material. These radioactive waste treatment systems were evaluated in the Final Environmental Statement (FES) dated February 1982 (NUREG-0841). The proposed increase in the capacity of the SFPs and the proposed increase in the enrichment limit will not involve any change in the waste treatment systems described in the FES.

Gaseous Radioactive Wastes

The storage of additional and higher enriched spent fuel assemblies in the pools is not expected to affect the releases of radioactive gases from the SFPs. Gaseous fission products such as krypton-85 and iodine-131 are produced by the fuel in the core during reactor operation. A small percentage of these fission gases is released to the reactor coolant from the small number of fuel assemblies that are expected to develop leaks during reactor operation. During refueling operations, some of these fission products enter the pools and are subsequently released into the air. Since the frequency of refueling (and, therefore, the number of freshly offloaded spent fuel assemblies stored in the pools at any one time) will not increase, there would be no increase in the amounts of these types of fission products released to the atmosphere as a result of the increased pool fuel storage capacity.

The increased heat load on the pools from the storage of additional spent fuel assemblies would potentially result in an increase in the pools' evaporation rate. However, this increased evaporation rate is not expected to result in an increase in the amount of gaseous tritium released from the pool. The overall release of radioactive gases from Palo Verde would remain a small fraction of the limits of 10 CFR 20.1301.

Solid Radioactive Wastes

Spent resins are generated by the processing of SFP water through the pools' purification system. These spent resins are disposed of as solid radioactive waste. Resin replacement is determined primarily by the requirement for water clarity and is normally done approximately once per year. No significant increase in the volume of solid radioactive waste is