

**NATIONAL SCIENCE FOUNDATION****Special Emphasis Panel in  
Mathematical Sciences; Notice of  
Meeting**

In accordance with the Federal Advisory Committee Act (Pub. L. 92-463, as amended), the National Science Foundation announced the following meeting.

*Name:* Special Emphasis Panel in Mathematical Sciences (1204).

*Date/Time:* February 17-19, 2000, 8 am-5 pm.

*Place:* National Science Foundation, 4201 Wilson Boulevard, Arlington, VA.

*Type of Meeting:* Closed.

*Contact Person:* Hans Engler, Program Director, Applied Mathematics Program, Room 1025, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230. Telephone: (703) 306-1870.

*Minutes:* May be obtained from the contact person listed above.

*Purpose of Meeting:* To provide advice and recommendations concerning proposals submitted to NSF for financial support.

*Agenda:* To review and evaluate proposals concerning Materials and Mechanic Research in the Mathematical Sciences as part of the selection process for awards.

*Reason for Closing:* The proposals being reviewed include information of a proprietary or confidential nature, including technical information; financial data, such as salaries and personal information concerning individuals associated with the proposals. These matters are exempt under 5 U.S.C. 552b(c) (4) and (6) of the Government in the Sunshine Act.

Dated: January 7, 2000.

**Karen J. York,**

*Committee Management Officer.*

[FR Doc 00-834 Filed 1-12-00; 8:45 am]

**BILLING CODE 7555-01-M**

**NUCLEAR REGULATORY  
COMMISSION****Indiana Michigan Power Company**

[Docket Nos. 50-315 and 50-316]

**Notice of Consideration of Issuance of  
Amendment to Facility Operating  
License, Proposed No Significant  
Hazards Consideration Determination,  
and Opportunity for a Hearing**

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. DPR-58 and DPR-74 issued to Indiana Michigan Power Company (the licensee) for operation of the Donald C. Cook Nuclear Power Plant, Units 1 and 2, located in Berrien County, Michigan.

The proposed amendments would delete the Donald C. Cook (D.C. Cook),

Unit 1 and 2, Technical Specification (TS) 5.4.2, "Reactor Coolant System Volume," because the information regarding the reactor coolant system (RCS) is not required by TS Section 5.0, "Design Features," for compliance with 10 CFR 50.36(c)(4). Changes to the RCS volume information are included in the D.C. Cook Updated Final Safety Analyses Report (UFSAR), and are controlled in accordance with 10 CFR 50.59.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Do the changes involve a significant increase in the probability of occurrence or consequences of an accident previously evaluated?

The proposed change to remove this information from T/S does not affect any accident initiators or precursors. Elimination of the RCS volume information from the T/S does not change the methods for plant operation or actions to be taken in the event of an accident. The quantity of radioactive material available for release in the event of an accident is not increased. Barriers to release of radioactive material are not eliminated or otherwise changed. More detailed and complete RCS component and piping volume information is included in the CNP [Cook Nuclear Plant] UFSAR, and changes to that information would be evaluated prior to implementation in accordance with 10 CFR 50.59. In addition, the proposed administrative format changes do not affect any of the technical content of the T/S.

Therefore, there is no significant increase in the probability of occurrence or consequences of an accident previously evaluated.

2. Do the changes create the possibility of a new or different kind of accident from any accident previously evaluated?

The deletion of the RCS volume information from the T/S does not change the methods of plant operation or modify plant systems, structures, or components. No new

methods of plant operation are created. As such, the proposed change does not affect any accident initiators or precursors or create new accident initiators or precursors. More detailed and complete RCS component and piping volume information is included in the CNP UFSAR, and any changes to that information would be evaluated prior to implementation in accordance with 10 CFR 50.59. In addition, the proposed administrative format changes do not affect any of the technical content of the T/S.

Therefore, the proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Do the changes involve a significant reduction in a margin of safety?

The deletion of the RCS volume information from the T/S does not affect safety limits or limiting safety system settings. Plant operational parameters are not affected. The proposed change does not modify the quantity of radioactive material available for release in the event of an accident. As such, the change will not affect any previous safety margin assumptions or conditions. The actual volume of the RCS is not affected by the change, only the location of the text describing the volume. More detailed and complete RCS component and piping volume information is included in the CNP UFSAR, and any changes to that information would be evaluated prior to implementation in accordance with 10 CFR 50.59. In addition, the proposed administrative format changes do not affect any of the technical content of the T/S.

Therefore, the proposed change does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92 are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days of the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will