

2. Is the burden estimate accurate?

3. Is there a way to enhance the quality, utility, and clarity of the information to be collected?

4. How can the burden of the information collection be minimized, including the use of automated collection techniques or other forms of information technology?

A copy of the draft supporting statement may be viewed free of charge at the NRC Public Document Room, 2120 L Street, NW, (Lower Level), Washington, DC. OMB clearance requests are available at the NRC worldwide web site (<http://www.nrc.gov/NRC/PUBLIC/OMB/index.html>). The document will be available on the NRC home page site for 60 days after the signature date of this notice.

Comments and questions about the information collection requirements may be directed to the NRC Clearance Officer, Brenda Jo. Shelton, U.S. Nuclear Regulatory Commission, T-6 F33, Washington, DC 20555-0001, by telephone at 301-415-7233 or by Internet electronic mail at BJS1@NRC.GOV.

Dated at Rockville, Maryland, this 23rd day of March, 1999.

For the U. S. Nuclear Regulatory Commission.

Brenda Jo. Shelton,

NRC Clearance Officer, Office of the Chief Information Officer.

[FR Doc. 99-7596 Filed 3-26-99; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket Nos. STN 50-454, STN 50-455, STN 50-456 and STN 50-457]

Commonwealth Edison Company; Notice of Consideration of Issuance of Amendments To Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. NPF-37, NPF-66, NPF-72, and NPF-77 issued to Commonwealth Edison Company (ComEd, the licensee) for operation of the Byron Station, Unit Nos. 1 and 2, located in Ogle County, Illinois, and Braidwood Station, Unit Nos. 1 and 2, located in Will County, Illinois.

The proposed amendments would allow the use of the Gamma-Metrics Post Accident Neutron Monitors (PANMs) to provide neutron flux

information during Operational Mode 6 (refueling).

Before issuance of the proposed license amendments, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendments requested involve no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendments would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

The installed Westinghouse source range neutron flux monitors are boron trifluoride detectors operating in the proportional region of the gas filled detector characteristic curve. The detectors monitor the neutron flux in counts per second. The instrument range covers six decades (i.e., 1 to 1E06 counts per second (cps)). The installed source range Gamma-Metrics post accident neutron flux monitors (PANMs) are enriched U-235 fission chambers operating in the ion chamber region of the detector characteristic curve. The detectors monitor the neutron flux in counts per second. The instrument range covers six decades (i.e., 0.1 to 1E05 cps). The detectors provide continuous visual indication in the Main Control Room (MCR.) Both the Westinghouse and Gamma-Metrics PANM neutron detectors are designed in accordance with 10CFR50 Appendix A, General Design Criterion (GDC) 13, "Instrumentation and control," and GDC 29, "Protection against anticipated operational occurrences," and are functionally equivalent for this application.

The Technical Specifications Bases state that two operable source range neutron flux monitors are required to provide a signal to alert the operator to unexpected changes in core reactivity such as with a boron dilution accident or an improperly loaded fuel assembly. The use of source range neutron flux monitors, either Gamma-Metrics PANMs or Westinghouse, has no effect on the probability of a dilution accident or an improperly loaded fuel assembly, because the source range neutron flux monitors are not in any way the initiators of or precursor to either accident.

The use of source range neutron flux monitors, either Gamma-Metrics PANMs or Westinghouse, has no effect on the

consequences of a dilution accident or an improperly loaded fuel assembly. The need for a safety analysis for an uncontrolled boron dilution accident is eliminated by isolating all unborated water sources as required by LCO 3.9.2, "Unborated Water Source Isolation Valves." Thus boron dilution is not considered a credible accident during refueling.

UFSAR Section 15.4.7 does not credit the source range neutron flux monitors for prevention or detection of the improperly loaded fuel assembly. It instead credits administrative procedures (i.e., nuclear component transfer lists, core inventory verification, etc.) implemented during fuel loading. If a fuel assembly loading error occurred, it would be detected by a flux map, or the perturbations of the power distribution will be sufficiently small to be within the allowable uncertainties.

Therefore this proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

The proposed change to Technical Specification 3.9.3 does not involve any physical alteration of plant systems, structures, or components, or changes in parameters governing plant operation. This change will not result in a significant reduction in monitoring capability since both the Westinghouse and Gamma-Metrics PANM source range neutron flux monitors are functionally equivalent and both are Safety Category I (Class 1E) systems. These source range instrumentation systems are for monitoring neutron flux and criticality assessment. They are not relied upon to initiate automatic accident mitigation in Operational Mode 6. Therefore, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

Does the proposed change involve a significant reduction in a margin of safety?

The proposed change will still maintain the requirement for two source range neutron flux monitors for visual monitoring of core reactivity as currently discussed in the Bases for the affected Technical Specifications. The Gamma-Metrics PANMs use fission chambers as detectors that have a sensitivity of 4 cps/neutron-volt (cps/nv) for thermal neutrons and 2 cps/nv for fast neutrons. The Westinghouse source range neutron flux monitors have a sensitivity of 13 cps/nv. The Gamma-Metrics PANMs have a comparable range and accuracy (i.e., range of 0.1 to 1E05 cps with an accuracy of 2% of full scale) to that of the Westinghouse source range neutron flux monitors (i.e., 1 to 1E+6 cps with an accuracy of 3 percent of full scale). The fact that the Gamma-Metrics PANMs do not cover the range of 1E05 to 1E06 cps is insignificant in Operational Mode 6 due to the low count rates expected. Therefore, these changes do not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three

standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendments requested involve no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendments until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendments before the expiration of the 30-day notice period, provided that its final determination is that the amendments involve no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the **Federal Register** a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this **Federal Register** notice. Written comments may also be delivered to Room 6D59, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By April 28, 1999, the licensee may file a request for a hearing with respect to issuance of the amendments to the subject facility operating licenses and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10

CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Byron Public Library District, 109 N. Franklin, P.O. Box 434, Byron, Illinois 61010 (for Byron) and the Wilmington Public Library, 201 S. Kankakee Street, Wilmington, Illinois 60481 (for Braidwood). If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to

rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendments under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendments requested involve no significant hazards consideration, the Commission may issue the amendments and make them immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendments.

If the final determination is that the amendments requested involve a significant hazards consideration, any hearing held would take place before the issuance of any amendments.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to Ms. Pamela B. Stroebel, Senior Vice President and General Counsel, Commonwealth Edison Company, P.O. Box 767, Chicago, Illinois 60690-0767, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests

for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendments dated March 22, 1999, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Byron Public Library District, 109 N. Franklin, P.O. Box 434, Byron, Illinois 61010 (for Byron) and the Wilmington Public Library, 201 S. Kankakee Street, Wilmington, Illinois 60481 (for Braidwood).

Dated at Rockville, Maryland, this 23rd day of March 1999.

For the Nuclear Regulatory Commission.
John B. Hickman,
*Project Manager, Project Directorate III-2,
 Division of Licensing Project Management,
 Office of Nuclear Reactor Regulation.*
 [FR Doc. 99-7601 Filed 3-26-99; 8:45 am]
 BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 72-20]

Department Of Energy, Idaho Operations Office Notice of Issuance of Materials License SNM-2508 for TMI-2 Independent Spent Fuel Storage Installation

The U.S. Nuclear Regulatory Commission (NRC or the Commission) has issued a Materials License under the provisions of Title 10 of the Code of Federal Regulations, Part 72 (10 CFR Part 72), to the Department of Energy, Idaho Operations Office (DOE-ID), authorizing receipt and storage of spent fuel in an independent spent fuel storage installation (ISFSI) located at the Idaho National Engineering and Environmental Laboratory (INEEL), within the Idaho Nuclear Technology and Engineering Center (INTEC) site in Scoville, Idaho, as described in its application dated October 31, 1996, and Safety Analysis Report (SAR).

The function of the ISFSI is to provide interim storage of radioactive material from the Three Mile Island Unit 2 (TMI-2) reactor core damaged by the March 28, 1979, reactor accident, including the remains of 177 Babcock and Wilcox 15x15 fuel assemblies, 61 control rod assemblies, and miscellaneous

irradiated core and core basket material. The material is contained within 265 fuel canisters, 12 knockout canisters, and 67 filter canisters which are used to confine the TMI-2 core debris in the absence of intact fuel assembly cladding. The cask that is authorized for use is the NUHOMS-12T designed by Transnuclear West, Inc. The license for an ISFSI under 10 CFR Part 72 is issued for 20 years, but the licensee may seek to renew the license, if necessary, prior to its expiration.

The Commission's Office of Nuclear Material Safety and Safeguards (NMSS) has completed its environmental, safeguards, and safety reviews in support of issuance of this license.

Following receipt of the application dated October 31, 1996, a "Notice of Consideration of Issuance of a Materials License for the Storage of Spent Fuel and Notice of Opportunity for a Hearing" was published in the **Federal Register** on January 13, 1997 (62 FR 1782). The "Final Environmental Impact Statement (FEIS) Related to the Construction and Operation of the TMI-2 Independent Spent Fuel Storage Installation," NUREG-1626, was issued and noticed in the **Federal Register** (63 FR 13077) on March 17, 1998, in accordance with 10 CFR Part 51. The scope of the FEIS included the construction and operation of an ISFSI on the INEEL site.

The staff has completed its safety review of the TMI-2 ISFSI site application and SAR. Materials License SNM-2508 and the NRC staff's "Safety Evaluation Report for the TMI-2 Independent Spent Fuel Storage Installation" were issued on March 19, 1999. Materials License SNM-2508, the staff's Environmental Impact Statement, Safety Evaluation Report, and other documents related to this action are available for public inspection and for copying for a fee at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC 20555, and at the Local Public Document Room at the INEEL Technical Library, 1776 Science Center Drive, Idaho Falls, ID 83402.

Dated at Rockville, Maryland, this 19th day of March 1999.

For the Nuclear Regulatory Commission.
E. William Brach,
*Director, Spent Fuel Project Office, Office of
 Nuclear Material Safety and Safeguards.*
 [FR Doc. 99-7600 Filed 3-26-99; 8:45 am]
 BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-275 and 50-323]

Pacific Gas and Electric Company; Notice of Consideration of Issuance of Amendments to Facility Operating Licenses and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. NPF-80 and 82, issued to Pacific Gas and Electric Company (PG&E or the licensee), for operation of the Diablo Canyon Power Plant, Units 1 and 2 (DCPP), located in San Luis Obispo County, California.

The initial notice of consideration of issuance of amendment to facility operating license and opportunity for hearing was originally published in the **Federal Register** (63 FR 55152) on October 14, 1998. The information included in the supplemental letters indicates that the original notice, that included 13 proposed beyond-scope issues (BSIs) to the Improved Technical Specifications (ITS) conversion, needs to be expanded (to add 15 new BSIs) and revised (to delete 8 previous BSIs) to include a total of 20 BSIs. This notice supersedes the previous notice.

The proposed amendment, requested by the licensee in a letter dated June 2, 1997, as supplemented by letters dated January 9, June 25, August 5, August 28, September 25, October 16, October 23, November 25, December 4, December 17, and December 30, 1998, and February 24 and March 10, 1999, would represent a full conversion from the current Technical Specifications (CTS) to a set of ITS based on NUREG-1431, "Standard Technical Specifications, Westinghouse Plants," Revision 1, dated April 1995. NUREG-1431 has been developed by the Commission's staff through working groups composed of both NRC staff members and industry representatives, and has been endorsed by the staff as part of an industry-wide initiative to standardize and improve the Technical Specifications (TSs) for nuclear power plants. As part of this submittal, the licensee has applied the criteria contained in the Commission's "Final Policy Statement on Technical Specification Improvements for Nuclear Power Reactors (Final Policy Statement)," published in the **Federal Register** on July 22, 1993 (58 FR 39132), to the CTS, and, using NUREG-1431 as a basis, proposed an ITS for DCPP. The criteria in the Final Policy Statement were subsequently added to 10 CFR 50.36, "Technical Specifications," in a