

For the Nuclear Regulatory Commission.

**Steven D. Bloom,**

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Division of Licensing Project Management,  
Office of Nuclear Reactor Regulation.*

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## NUCLEAR REGULATORY COMMISSION

[Docket No. 50-482]

### **Wolf Creek Nuclear Operating Corporation; Notice of Issuance of Amendment to Facility Operating License**

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment No. 122 to Facility Operating License No. NPF-42 issued to the Wolf Creek Nuclear Operating Corporation (WCNOC or the licensee), which revised the technical specifications for operation of the Wolf Creek Generating Station (WCGS) located in Coffey County, Kansas.

The amendment is effective as of the date of issuance.

The amendment revises Technical Specification (TS) 4.7.3b., "Plant Systems—Component Cooling Water System—Surveillance Requirements," by deleting the requirement to perform the specified surveillances during shutdown.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity for Hearing in connection with this action was published in the **Federal Register** on October 5, 1998 (63 FR 53471), February 26, 1999 (64 FR 9546), and March 1, 1999 (64 FR 10028). No request for a hearing or petition for leave to intervene was filed following these notices.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of this amendment will not have a significant effect on the quality of the human environment.

For further details with respect to the action see (1) the application for

amendment dated May 15, 1997, as supplemented by letters dated June 30, August 5, August 28, September 24, October 16, October 23, November 24, December 2, December 17, and December 21, 1998 and January 15, 1999, (2) Amendment No. 122 to Facility Operating License No. NPF-42, and (3) the Commission's related Safety Evaluation and Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document rooms located at the Emporia State University, William Allen White Library, 1200 Commercial Street, Emporia, Kansas 66801, and Washburn University School of Law Library, Topeka, Kansas 66621. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C., 20555, Attention: Director, Division of Licensing Project Management.

Dated at Rockville, Maryland, this 23rd day of March 1999.

For the Nuclear Regulatory Commission.

**Kristine M. Thomas,**

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Office of Nuclear Reactor Regulation.*

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## NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-269, 50-270, and 50-287]

### **Duke Energy Corporation; Oconee Nuclear Station, Units 1, 2, and 3 Environmental Assessment and Finding of No Significant Impact**

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from the requirements of Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Section 50.60 and Appendix G to the Duke Energy Corporation (the licensee) for operation of the Oconee Nuclear Station, Units 1, 2, and 3, located in Oconee County, South Carolina.

#### **Environmental Assessment**

##### *Identification of the Proposed Action*

The proposed action would exempt the licensee from the provisions in 10 CFR Part 50, Section 50.60 and Appendix G. The NRC has established requirements in 10 CFR Part 50 to protect the integrity of the reactor coolant pressure boundary (RCPB) in

nuclear power plants. As part of these requirements, 10 CFR Part 50, Appendix G requires that pressure-temperature (P-T) limits be established for reactor pressure vessels (RPVs) during normal operating and hydrostatic, or leak rate, testing conditions. Specifically, 10 CFR Part 50, Appendix G states that "[t]he appropriate requirements on \* \* \* the pressure-temperature limits and minimum permissible temperature must be met for all conditions." Pressurized water reactor licensees have installed cold overpressure mitigation systems/low temperature overpressure protection (LTOP) systems in order to protect the RCPBs from being operated outside of the boundaries established by the P-T limit curves and to provide pressure relief of the RCPBs during low temperature overpressurization events. The licensee is required by the Oconee Units 1, 2, and 3 Technical Specifications (TSs) to update and submit the changes to its LTOP setpoints whenever the licensee is requesting approval for amendments to the P-T limit curves in the Oconee Units 1, 2, and 3 TSs.

As a result, to approve its amendments to the TS P-T limit curves, the licensee requested in its submittal dated October 15, 1998, that the staff exempt Oconee Units 1, 2, and 3 from the application of specific requirements of 10 CFR Part 50, Section 50.60 and Appendix G and substitute use of the American Society of Mechanical Engineers (ASME) Code Case N-514, "Low Temperature Overpressure Protection Section XI, Division 1." This would permit setting the pressure setpoint of the facility's LTOP such that the P-T limits required by 10 CFR Part 50, Appendix G could be exceeded by 10 percent during a low temperature pressure transient. The submittal was supplemented by letters dated December 15, 1998, and January 11 and 21, 1999.

##### *The Need for the Proposed Action*

The licensee has noted in its submittal of October 15, 1998, that the underlying purpose of the regulations is to establish limits to protect the RPVs from brittle failure during low temperature operation and that the LTOP provides a physical means of protecting these limits. As a means of determining the LTOP enable temperature, the licensee proposed to use the ASME Code Case N-514 to permit setting the pressure setpoint of the facility's LTOP such that the P-T limits required by 10 CFR Part 50, Appendix G could be exceeded by 10 percent during a low temperature pressure transient. The use of this Code