allow for improved system operability by lowering the setpoint from being set too close to the expected pressure measured at the particular time of the heating evolution. The revised methodology allows for the control system to accommodate for instrument uncertainty when determining the actual setpoint (a procedurally controlled value). The revised allowable value of 15 psia still allows for a 50% rise in pressure from the administratively controlled starting pressure of 10 psia, which allows for the low cylinder pressure control system to suitably demonstrate line clarity.

The change from the current surveillance requirements specifying ''. . . after the first hour of heating . . .'' to the proposed allowable value of less than or equal to 65 minutes will not lead to a significant increase in measured pressure. The proposed value provides a definitive time for the surveillance requirements to be performed within an expected pressure range that is well below the maximum allowable working pressure of the UF_6 cylinder.

The changes will not result in a significant reduction in any margin of safety.

7. The proposed amendment will not result in an overall decrease in the effectiveness of the plant's safety, safeguards, or security programs.

The proposed amendment which involves changing the surveillance requirements and actuation setpoints of the low cylinder pressure shutoff will not affect system operability. The low cylinder pressure shutoff system will still be capable of ensuring UF_6 process line clarity and the availability of the UF_6 cylinder high pressure autoclave steam shutdown system. Therefore, the proposed modifications will not result in a decrease in the overall effectiveness of the plant's safety program. The staff has also not identified any safeguards or security related implications from the proposed amendment.

Effective date: 60 days after issuance of amendment.

Certificate of Compliance No. GDP–2: Amendment will revise the Technical Safety Requirement.

Local Public Document Room location: Portsmouth Public Library, 1220 Gallia Street, Portsmouth, Ohio 45662.

Dated at Rockville, Maryland, this 27th day of October 1998.

Carl J. Paperiello,

Director, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 98–29493 Filed 11–3–98; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Dockets 70–7001 and 70–7002]

Notice of Amendment to Certificates of Compliance GDP–1 and GDP–2 for the U.S. Enrichment Corporation (Paducah Gaseous Diffusion Plant and Portsmouth Gaseous Diffusion Plant, Paducah, Kentucky and Piketon, Ohio)

The Director, Office of Nuclear Material Safety and Safeguards, has made a determination that the following administrative amendments are not significant in accordance with 10 CFR 76.45. In making that determination, the staff concluded that: (1) There is no change in the types or significant increase in the amounts of any effluents that may be released offsite; (2) there is no significant increase in individual or cumulative occupational radiation exposure; (3) there is no significant construction impact; (4) there is no significant increase in the potential for, or radiological or chemical consequences from, previously analyzed accidents; (5) the proposed changes do not result in the possibility of a new or different kind of accident; (6) there is no significant reduction in any margin of safety; and (7) the proposed changes will not result in an overall decrease in the effectiveness of the plant's safety, safeguards or security programs. The basis for this determination for the amendments is shown below.

The NRC staff has concluded that the administrative amendments provide reasonable assurance of adequate safety, safeguards, and security, and compliance with NRC requirements. Therefore, the Director, Office of Nuclear Material Safety and Safeguards, is prepared to issue amendments to the Certificates of Compliance for the Paducah Gaseous Diffusion Plant and the Portsmouth Gaseous Diffusion Plant. The staff has prepared a Compliance Evaluation Report which provides details of the staff's evaluation.

The NRC staff has determined that these amendments satisfy the criteria for a categorical exclusion in accordance with 10 CFR 51.22(c)(19). Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared for these amendments.

USEC or any person whose interest may be affected may file a petition, not exceeding 30 pages, requesting review of the Director's Decision. The petition must be filed with the Commission not later than 15 days after publication of this **Federal Register** notice. A petition for review of the Director's Decision shall set forth with particularity the interest of the petitioner and how that interest may be affected by the results of the decision. The petition should specifically explain the reasons why review of the Decision should be permitted with particular reference to the following factors: (1) The interest of the petitioner; (2) how that interest may be affected by the Decision, including the reasons why the petitioner should be permitted a review of the Decision; and (3) the petitioner's areas of concern about the activity that is the subject matter of the Decision. Any person described in this paragraph (USEC or any person who filed a petition) may file a response to any petition for review, not to exceed 30 pages, within 10 days after filing of the petition. If no petition is received within the designated 15-day period, the Director will issue the final amendment to the Certificate of Compliance without further delay. If a petition for review is received, the decision on the amendment application will become final in 60 days, unless the Commission grants the petition for review or otherwise acts within 60 days after publication of this **Federal Register** notice.

A petition for review must be filed with the Secretary, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC, by the above date.

For further details with respect to the action see the Commission's Compliance Evaluation Report.

Date of amendment request: There was no formal amendment request. USEC, Inc. was formally privatized by the U.S. Treasury Department on July 28, 1998. These amendments will impose the conditions of transfer attached to the privatization action.

Brief description of amendment: The Commission conditionally consented to the transfer of the certificates of compliance to the privatized USEC. This transfer was subject to USEC consenting to three proposed conditions that related to foreign ownership, control or domination and the Russian High Enriched Uranium Agreement. On July 20, 1998, USEC consented to the conditions. USEC, Inc. was formally privatized by the U.S. Treasury Department on July 28, 1998. These amendments do not approve the transfer; they are administrative amendments to add the conditions of transfer to the certificates of compliance.

Basis for finding of no significance: 1. The proposed amendment will not result in a change in the types or significant increase in the amounts of any effluents that may be released offsite.

Operations at the gaseous diffusion plants will remain unchanged. The proposed action is to add three conditions of transfer to the certificates of compliance. There will be no change to the types or amount of effluents that may be released offsite.

2. The proposed amendment will not result in a significant increase in individual or cumulative occupational radiation exposure.

The proposed amendments will not increase any exposure to radiation. The action is solely an administrative action. Therefore, the changes will not result in a significant increase in individual or cumulative radiation exposure.

3. The proposed amendment will not result in a significant construction impact.

The proposed amendment will not result in any construction; therefore, there will be no construction impacts.

4. The proposed amendment will not result in a significant increase in the potential for, or radiological or chemical consequences from, previously analyzed accidents.

The proposed administrative amendments to add the conditions of transfer to the certificates of compliance have no impact on the potential for or occurrence of an accident or the resulting consequences. Therefore, these changes will not increase the probability of occurrence or consequence of any postulated accident currently identified in the safety analysis reports.

5. The proposed amendment will not result in the possibility of a new or different kind of accident.

The proposed amendments will not create the possibility of a new or different type of equipment malfunction or a new or different type of accident.

6. The proposed amendment will not result in a significant reduction in any margin of safety.

The proposed amendments to add conditions to the certificates of compliance will have no impact on the margin of safety. Therefore, these changes do not decrease the margins of safety.

7. The proposed amendment will not result in an overall decrease in the effectiveness of the plant's safety, safeguards or security programs.

Implementation of the proposed amendments does not change the safety, safeguards, or security programs. Therefore, the effectiveness of the safety, safeguards, and security programs is not decreased. In approving the transfer, the Commission determined that the transfer would not be inimical to the common defense and security of the United States.

Effective date: The amendments to Certificates of Compliance GDP–1 and GDP–2 become effective immediately upon being signed by the Director, Office of Nuclear Material Safety and Safeguards.

Certificates of Compliance GDP–1 and GDP–2: Amendments will revise the certificates of compliance to add three conditions of transfer related to foreign ownership, control or domination and the Russian High Enriched Uranium Agreement.

Local Public Document Room locations: Paducah Public Library, 555 Washington Street, Paducah, Kentucky 42003 and Portsmouth Public Library, 1220 Gallia Street, Portsmouth, Ohio 45662.

Dated at Rockville, Maryland, this 27th day of October, 1998.

For the Nuclear Regulatory Commission.

Carl J. Paperiello,

Director, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 98–29494 Filed 11–3–98; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards Subcommittee Meeting on Plant License Renewal

The ACRS Subcommittee on Plant License Renewal will hold a meeting on November 18, 1998, Room T–2B3, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Wednesday, November 18, 1998—8:30 a.m. until the conclusion of business.

The Subcommittee will discuss generic issues related to license renewal activities and the schedule for their resolution, selected technical and topical reports and the associated safety evaluation reports, status of the staff's review of the applications for renewing licenses for Calvert Cliffs and Oconee Nuclear Power Plants, and related matters. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittee, along with any of its consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Subcommittee will then hear presentations by and hold discussions with representatives of the NRC staff, its consultants, and other interested persons regarding this review.

Further information regarding topics to be discussed, whether the meeting has been canceled or rescheduled, and the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor can be obtained by contacting the cognizant ACRS staff engineer, Dr. Medhat El-Zeftawy (telephone 301/415–6889) between 7:30 a.m. and 4:15 p.m. (EST). Persons planning to attend this meeting are urged to contact the above named individual one or two working days prior to the meeting to be advised of any potential changes in the proposed agenda, etc., that may have occurred.

Dated: October 29, 1998.

Sam Duraiswamy,

Chief, Nuclear Reactors Branch.

[FR Doc. 98–29499 Filed 11–3–98; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards Subcommittee Meeting on Thermal-Hydraulic Phenomena

The ACRS Subcommittee on Thermal-Hydraulic Phenomena will hold a meeting on November 19, 1998, Room T–2B3, 11545 Rockville Pike, Rockville, Maryland.

A portion of this meeting will be closed to public attendance to discuss Westinghouse proprietary information pursuant to 5 U.S.C. 552b(c)(4).

The agenda for the subject meeting shall be as follows:

Thursday, November 19, 1998—8:30 a.m.—12:00 Noon.