The Need for the Proposed Action

The proposed action is needed to support the progression to startup for WNP–2, which is currently in a refueling outage. During this outage newly designed suction strainers have been installed in the suppression pool. They are designed to protect ECCS pumps from fibrous or other material that could be transported to the suppression pool after a design basis accident such as a loss of coolant accident. The licensee determined after fabrication of these strainers that the stanless steel material had measured yield strength which exceeded the limit which was specified in the FSAR. Excessive yield strength can make the stainless steel susceptible to stress corrosion cracking (SCC) under certain environmental conditions. The licensee identified this as an unreviewed safety issue and submitted an amendment request which would change the yield strength for the installed strainers. Approval of this amendment will enable the licensee to change reactor mode and declare the strainers operable while progressing to startup and full power operation.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed action and, based on the testing and analytical information provided by the licensee, concludes that the increase in yield strength for the specific material used in the suction strainers is acceptable. The licensee has an effective cleanup system for the suppression pool, which maintains a desired level of water cleanliness sufficient to avoid conditions that would support SCC. Further, the licensee has conducted a fracture mechanics analysis and has determined that cracking in the surface martensitic structure of the strainers will not propagate to a critical size and, thus, not jeopardize the strainers' safety related function of protecting the ECCS pumps and spray nozzles. Also, the licensee's analysis has demonstrated that the strainers have adequate structural integrity to preclude failure when the forces of design basis hydrodynamic loads are applied. Lastly, a Strauss test using actual strainer material samples demonstrated acceptable stress corrosion cracking resistance.

The staff has concluded that this change will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released offsite, and there is no significant increase in the allowable offsite or occupational radiation exposure. Accordingly, the Commission concludes that there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does not affect nonradiological plant effluents and has no other environmental impact.

Accordingly, the Commission concludes that there are no significant environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

Since the Commission has concluded there is no significant environmental impact associated with the proposed action, any alternatives with equal or greater environmental impact need not be evaluated. As an alternative to the proposed action, the staff considered denial of the proposed action. Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for WNP–2.

Agencies and Persons Consulted

In accordance with its stated policy, on May 13, 1998, the staff consulted with the Washington State official, Mr. R. Cowley of the Department of Health, State of Washington Energy Facility Site Evaluation Council, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated April 16, 1998, as supplemented by letters dated April 28, 1998, and May 8, 1998, which are available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Richmond Public Library, 955 Northgate Street, Richland, Washington 99352.

Dated at Rockville, MD., this 14th day of May 1998.

For the Nuclear Regulatory Commission.

Chester Poslusny,

Senior Project Manager, Project Directorate IV–2, Division of Reactor Projects—III/IV, Office of Nuclear Reactor Regulation.

[FR Doc. 98–13504 Filed 5–20–98; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards, Subcommittee Meeting on Thermal-Hydraulic Phenomena; Notice of Meeting

The ACRS Subcommittee on Thermal-Hydraulic Phenomena will hold a meeting on June 11–12, 1998, Room T–2B1, 11545 Rockville Pike, Rockville, Maryland.

Portions of the meeting will be closed to public attendance to discuss Westinghouse Electric Company proprietary information pursuant to 5 U.S.C. 552b(c)(4).

The agenda for the subject meeting shall be as follows:

Thursday, June 11, 1998—8:30 a.m. until the conclusion of business.

Friday, June 12, 1998—8:30 a.m. until the conclusion of business.

The Subcommittee will continue its review of the Westinghouse AP600 Test and Analysis Program (TAP) in support of the AP600 design certification. During this meeting, the Subcommittee will focus its review on the issues associated with the Westinghouse TAP for the Passive Containment System, including those identified in the February 19, 1998 ACRS letter to the NRC Executive Director for Operations. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittee, along with

any of its consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Subcommittee will then hear presentations by and hold discussions with representatives of the Westinghouse Electric Company, the NRC staff, their consultants, and other interested persons regarding this review.

Further information regarding topics to be discussed, whether the meeting has been canceled or rescheduled, the scheduling of sessions which are open to the public, and the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor, can be obtained by contacting the cognizant ACRS staff engineer, Mr. Paul A. Boehnert (telephone 301/415–8065) between 7:30 a.m. and 4:15 p.m. (EDT). Persons planning to attend this meeting are urged to contact the above named individual one or two working days prior to the meeting to be advised of any potential changes to the agenda, etc., that may have occurred.

Dated: May 14, 1998.

Sam Duraiswamy,

Chief, Nuclear Reactors Branch.

[FR Doc. 98–13472 Filed 5–20–98; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–271]

Vermont Yankee Nuclear Power Corporation, Vermont Yankee Nuclear Power Station; Receipt of Petition for Director's Decision Under 10 CFR 2.206

Notice is hereby given that by Petition dated April 9, 1998, Mr. Michael J. Daley, on behalf of the New England Coalition on Nuclear Pollution, Inc. (or Petitioner), requested that the U.S. Nuclear Regulatory Commission (NRC) take immediate action with regard to the Vermont Yankee Nuclear Power Station. The Petitioner requests that the NRC issue an order requiring that the licensee's administrative limits, which preclude Vermont Yankee Nuclear Power Station from operating with a torus water temperature above 80 °F or with service water injection temperature greater than 50 °F, shall remain in force until certain conditions are met. The requested conditions include a complete reconstitution of the licensing basis for the maximum torus water temperature, submittal to the NRC of a technical specifications amendment request establishing the correct maximum torus water temperature, and completion of NRC review of the amendment request.

As the basis for this request, the Petitioner states that the licensee has been unable to demonstrate an ability to either justify the operational limits for the maximum torus water temperature or maintain operations within existing administrative limits (torus water temperature is critical to the proper functioning of the containment). The Petitioner also states that the NRC must move from a ''wait and see'' posture to active intervention, with immediate imposition of the order as a necessary first step.

The request is being treated pursuant to 10 CFR 2.206 of the Commission's regulations. The request has been referred to the Director of the Office of Nuclear Reactor Regulation. As provided by § 2.206, appropriate action will be taken on this petition within a reasonable time.

By letter dated May 13, 1998, the Director denied Petitioner's request for immediate action at Vermont Yankee Nuclear Power Station.

A copy of the petition is available for inspection at the Commission's Public Document Room at 2120 L Street, NW., Washington, DC. 20555–0001 and at the local public document room located at Brooks Memorial Library, 224 Main Street, Brattleboro, VT 05301.

Dated at Rockville, MD, this 13th day of May, 1998.

For the Nuclear Regulatory Commission.

Samuel J. Collins,

Director, Office of Nuclear Reactor Regulation.

[FR Doc. 98–13508 Filed 5–20–98; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No.: 040–07982]

Consideration of Amendment Request To Approve a Decommissioning Plan for Alliant Techsystems, Inc., and Opportunity for a Hearing

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of Intent to approve decommissioning plan license amendment and opportunity for Hearing related to source materials license for Alliant Techsystems, Inc.

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of a license amendment to Source Material License No. SUB–971, issued to Alliant Techsystems, Inc., to authorize decontamination and decommissioning activities of those areas of the licensee's Twin Cities Army Ammunition Plant, Depleted Uranium Facilities, New Brighton, Minnesota, site which require remediation prior to release for unrestricted use.

The licensee requested the amendment in a letter dated October 6, 1997. The amendment would incorporate the licensee's *Decommissioning Plan for the Twin Cities Army Ammunition Plant, Depleted Uranium Facilities, New Brighton, Minnesota.* The plan discusses the administrative and technical procedures necessary for performing the decommissioning project as follows: (1) Summary of Plan (including background, description of facilities to be remediated, etc.); (2) Choices of Decommissioning Alternatives and Decommissioning Activities (including decommissioning schedule, organization and program responsibilities); (3) Protection of Occupational and Public Health and Safety (including radiation protection, asbestos protection and waste management programs); (4) Final Radiation Safety Survey; (5) Decommissioning Cost Estimate and Funding Plan; (6) Decommissioning Quality Assurance Plan; and (7) References and Appendices.

The NRC will require the licensee to remediate the Depleted Uranium facilities to meet NRC's decommissioning criteria, and during the decommissioning activities, to maintain effluents and doses within NRC requirements and as low as reasonably achievable.

Prior to approving the decommissioning plan, NRC will have made findings required by the Atomic Energy Act of 1954, as amended, and NRC's regulations. Staff review findings and approval of the plan will be documented in an amendment to License No. SUB–971.

The NRC hereby provides notice that this is a proceeding on an application for a license amendment falling within the scope of Subpart L, Informal Hearing Procedures for Adjudications in Materials Licensing Proceedings, of the NRC's rules of practice for domestic licensing proceedings in 10 CFR part 2. Pursuant to § 2.1205(a), any person whose interest may be affected by this proceeding may file a request for a hearing in accordance with § 2.1205(d). A request for a hearing must be filed within thirty (30) days of the date of publication of this **Federal Register** Notice. The request for a hearing must be filed with the Office of the Secretary either: