

containment as a result of a lube oil leak from the RCPs. The ROALs are a low pressure system. The 3000 psi minimum design pressure of the ROALs is significantly higher than the 30 psig line operating pressures. All piping components associated with the suction and discharge of the portable oil transfer pump skid are appropriately rated for the service conditions. The hose connections are flexible hoses, and therefore, are not subjected to mechanical vibration and thermal stresses.

Oil leakage from the ROALs is not expected to occur during oil transfer operations. The ROALs are used only periodically and operated using controlled procedures and processes. The controlled oil addition process includes determining the amount of oil to be added, performing a walk down before oil addition to check for leakage, and local and control room monitoring of the oil addition process. Following the addition of the proper amount of oil, the ROAL is drained either by gravity or by reversing the pump suction and discharge connections and pumping down the line. The upper and lower reservoirs contain only limited quantities of oil, 175 and 10 gallons, respectively. Based on the maximum oil addition allowed by procedures, the maximum potential oil spill will be only 12 gallons.

During power operation, damage to the ROALs would not occur because the reactor building access and work activities are limited during this time. Further, following refueling outages, containment close-out procedures require visual inspections to assure the integrity of the ROALs.

Inside the D-ring, the ROALs travel over or along hot components that are insulated with a non-absorbing material or encased mineral wool. The surface temperatures of the insulation are below the ignition temperature of the oil, such that the insulation would not be a potential ignition source. The construction of this insulation makes it less likely for potential leaking oil to soak the encased mineral wool. Any potential oil leak in this area would be reasonably expected to travel down the insulation to the floor. Further, there are spot-type heat detectors located in this general area which can provide early warning to the control room in the event of a fire.

Outside the secondary shield wall, the ROALs do not travel over any hot main coolant pipes or steam lines and any potential leak in this area would pool on the floor and have no opportunity for ignition.

The ROALs are routed through two fire areas in the reactor building (RB), elevations 95 and 119 feet, designated as RB-95-300 and RB-119-302. The licensee has administrative controls that are designed to control the type, amount, use and location of combustibles. Proper control of combustibles minimizes the possibility of starting, spreading, or contributing to a fire. The probability for a fire hazard in this area is minimal because of separation of redundant components, the surrounding concrete structure, minimal or no intervening combustibles in the area, high ROAL design pressure and low operating pressure, and the short duration and infrequency of oil addition operations. However, the licensee has evaluated a worst case lube oil fire for these fire areas that contain ROALs and concluded that it is bounded by the CR3's existing Fire Hazards Analysis and Appendix R fire study.

In 1985, when the ROALs were originally installed, they were functionally leak tested with no visible leakage detected. During the last 12 years of performance there has been no indication of any leakage from the ROALs.

Fire detection and manual fire suppression equipment is available in the vicinity of the lube oil fill lines. In the event of a fire, it is expected that a detector will alarm while the fire is in its incipient stages. Operators would then take appropriate action to mitigate the consequences of the fire. This provides further assurance that a worst-case postulated fire would not damage safe shutdown equipment.

Based on the design features of the ROALs and associated lube oil collection systems, and the licensee's proposed compensatory actions, there is reasonable assurance that the RCP lube oil system will not lead to a major fire hazard. In addition, based on the present level of fire protection provided for the RCPs, if a fire were to occur in the area, there is reasonable assurance that the fire will be detected and mitigated. Therefore, the staff finds the ROALs without an oil collection system acceptable.

The underlying purpose of 10 CFR part 50, Appendix R, Section III.O, is to ensure that lube oil from all potential pressurized and unpressurized leakage sites in the reactor coolant pump lube oil systems would be collected and not become a fire hazard such that "the capability to achieve and maintain safe shutdown conditions during and after any postulated fire in the plant" will be ensured. On the basis of its review and evaluation of the licensee's exemption

request, the staff concludes that the addition of an oil collection system for the ROALs is not necessary to achieve the underlying purpose of the rule. Therefore, an exemption from the requirement for providing a lube oil collection system for the RCP Motor ROALs is acceptable.

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For the foregoing reasons, the NRC staff has concluded that pursuant to 10 CFR 50.12(a) an exemption permitting the licensee's proposed use of ROALs without an oil collection system is authorized by law, will not present an undue risk to public health and safety and is consistent with the common defense and security. The NRC staff has determined that there are special circumstances present, as specified in 10 CFR 50.12(a)(2)(ii), in that application of 10 CFR part 50, Appendix R, Section III.O, is not necessary in order to achieve the underlying purpose of this regulation.

Accordingly, the Commission hereby grants an exemption from the technical requirements of 10 CFR part 50, Appendix R, Section III.O to the extent that the ROALs need not be provided with an oil collection system.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will not have a significant effect on the quality of the human environment (62 FR 59752).

This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 19th day of November 1997.

For the Nuclear Regulatory Commission.

Samuel J. Collins,

Director, Office of Nuclear Reactor Regulation.

[FR Doc. 97-31085 Filed 11-25-97; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket Number 40-6622]

Pathfinder Mines Corporation

AGENCY: U.S. Nuclear Regulatory Commission.

ACTION: Amendment of Source Material License SUA-442 to change three reclamation milestone dates.

SUMMARY: Notice is hereby given that the U.S. Nuclear Regulatory Commission has amended Pathfinder Mines Corporation's (PMC's) Source Material License SUA-442 to change three reclamation milestone dates. This amendment was requested by PMC in

its letter dated September 11, 1997, and the receipt of the request by NRC was noticed in the **Federal Register** on October 6, 1997.

The license amendment modifies License Condition 50 to change completion dates for three site-reclamation milestones. The new dates approved by the NRC extend completion of placement of the interim cover over tailings pile by two years, and completion of placement of the final radon barrier and placement of the erosion protection cover by three years. PMC attributes the delays to a substantial volume of water still remaining to be evaporated from the tailings system, before an interim cover could be placed. Based on the review of PMC's submittal, the NRC staff concludes that the delays are attributable to factors beyond the control of PMC, the proposed work is scheduled to be completed as expeditiously as practicable, and the added risk to the public health and safety is not significant.

An environmental assessment is not required since this action is categorically excluded under 10 CFR 51.22(c)(11), and an environmental

report from the licensee is not required by 10 CFR 51.60(b)(2).

SUPPLEMENTARY INFORMATION: PMC's amended license, and the NRC staff's technical evaluation of the amendment request are being made available for public inspection at the Commission's Public Document Room at 2120 L Street, NW. (Lower Level), Washington, DC 20555.

FOR FURTHER INFORMATION CONTACT: Mohammad W. Haque, Uranium Recovery Branch, Division of Waste Management, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Telephone (301) 415-6640.

Dated at Rockville, Maryland, this 19th day of November, 1997.

Joseph J. Holonich,

Chief, Uranium Recovery Branch, Division of Waste Management, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 97-31086 Filed 11-25-97; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Application for a License To Export Special Nuclear Material

Pursuant to 10 CFR 110.70(b) "Public notice of receipt of an application", please take notice that the Nuclear Regulatory Commission has received the following application for an export license. Copies of the application are on file in the Nuclear Regulatory Commission's Public Document Room located at 2120 L Street, N.W., Washington, D.C.

A request for a hearing or petition for leave to intervene may be filed within 30 days after publication of this notice in the **Federal Register**. Any request for hearing or petition for leave to intervene shall be served by the requester or petitioner upon the applicant, the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555; the Secretary, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555; and the Executive Secretary, U.S. Department of State, Washington, D.C. 20520.

The information concerning the application follows.

NRC EXPORT LICENSE APPLICATION

Name of applicant, date of application, date received, application No.	Description of material		End use	Country of origin
	Material type	Total quantity		
Transnuclear, Inc., October 27, 1997, October 29, 1997, XSNM03012.	High-enriched Uranium (93.3%)	26.738 kg ...	Fabrication of target material for production of medical isotopes.	Canada.

Dated this 12th day of November 1997 at Rockville, Maryland.

For the Nuclear Regulatory Commission.

Ronald D. Hauber,

Director, Division of Nonproliferation, Exports and Multilateral Relations, Office of International Programs.

[FR Doc. 97-30968 Filed 11-25-97; 8:45 am]

BILLING CODE 7590-01-M

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards Subcommittee Meeting on Planning and Procedures

The ACRS Subcommittee on Planning and Procedures will hold a meeting on December 3, 1997, Room T-2B1, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance, with the exception of a portion that may be closed pursuant

to 5 U.S.C. 552b(c) (2) and (6) to discuss organizational and personnel matters that relate solely to internal personnel rules and practices of ACRS, and information the release of which would constitute a clearly unwarranted invasion of personal privacy.

The agenda for the subject meeting shall be as follows:

Wednesday, December 3, 1997—10:00 a.m. until 11:30 a.m.

The Subcommittee will discuss proposed ACRS activities and related matters. It may also discuss the qualifications of candidates for appointment to the ACRS. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee

Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff person named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

Further information regarding topics to be discussed, the scheduling of sessions open to the public, whether the meeting has been canceled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements, and the time allotted therefor can be obtained by contacting the cognizant ACRS staff person, Dr. John T. Larkins (telephone: 301/415-7360) between 7:30 a.m. and 4:15 p.m. (EST). Persons planning to attend this