

such statements should be sent to the Executive Secretary of the Advisory Council at the above address. Papers will be accepted and included in the record of the meeting if received on or before October 30.

Signed at Washington, D.C. this 15th day of October, 1997.

**Olena Berg,**

*Assistant Secretary, Pension and Welfare Benefits Administration.*

[FR Doc. 97-27956 Filed 10-2-97; 8:45 am]

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## DEPARTMENT OF LABOR

### Pension and Welfare Benefits Administration

#### 100th Full Meeting of the Advisory Council on Employee Welfare and Pension Benefits Plans; Notice of Meeting

Pursuant to the authority contained in Section 512 of the Employee Retirement Income Security Act of 1974 (ERISA), 29 U.S.C. 1142, the 100th public meeting will be held November 13, 1997 of the Advisory Council on Employee Welfare and Pension Benefit Plans.

The session will take place in the Secretary's Conference Room S-2508, U.S. Department of Labor Building, Second and Constitution Avenue, NW, Washington, D.C. 20210. The purpose of the open meeting, which will run from 1:00 p.m. until approximately 2:30 p.m., is for working group chairs and vice chairs to present their groups' final reports and recommendations of the year to the full Council for its action on their findings and/or acceptance before the reports and recommendations are officially forwarded to the Secretary of Labor. The meeting also will provide the opportunity for an update on activities of the Pension and Welfare Benefits Administration by the Assistant Secretary of that organization and for a formal ceremony of appreciation for outgoing members of the Council.

Members of the public are encouraged to file a written statement pertaining to the Council's specific topics for the year by submitting 20 copies on or before October 30, 1997, to Sharon Morrissey, Executive Secretary, ERISA Advisory Council, U.S. Department of Labor, Room N-5677, 200 Constitution Avenue, NW, Washington, D.C. 20210. Individuals or representatives of organizations wishing to address the Council should forward their request to the Executive Secretary or telephone (202) 219-8753. Oral presentations will be limited to 10 minutes, time permitting, but an extended written

statement may be submitted for the record. Individuals with disabilities, who need special accommodations, should contact Sharon Morrissey by October 30, at the address indicated in this notice.

Organizations or individuals may also submit statements for the record without testifying. Twenty (20) copies of such statements should be sent to the Executive Secretary of the Advisory Council at the above address. Papers will be accepted and included in the record of the meeting if received on or before October 30.

Signed at Washington, D.C. this 15th day of October, 1997.

**Olena Berg,**

*Assistant Secretary, Pension and Welfare Benefits Administration.*

[FR Doc. 97-27957 Filed 10-21-97; 8:45 am]

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## NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-369 and 50-370]

### Duke Energy Corporation; Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. NPF-9 and NPF-17 issued to Duke Energy Corporation (the licensee) for operation of the McGuire Nuclear Station, Units 1 and 2, located in Mecklenburg County, North Carolina.

The proposed amendments would revise Technical Specification Table 3.3-4, "Engineered Safety Features [ESF] Actuation System Instrument Trip Setpoints." Specifically, the amendments would support the replacement of the three safety-related wide range level instruments. The ESF trip setpoint for the refueling water automatic switch over to recirculation would be revised to account for the difference in instrument uncertainty associated with wide range level instruments and provide additional operator response time margin.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no

significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

#### First Standard

Operation of the facility in accordance with the proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

#### Probability

The FWST [Refueling Water Storage Tank] and its associated instrumentation are not considered accident initiators. The instrumentation change is from a narrow range type instrument to a wide range type instrument. A failure of either type of instrument could result in an undesired switch over or failure to switchover. However, the failure could not initiate any subsequent accident sequences.

#### Consequences

With the switchover to recirculation setpoint change, the system design will still provide enough injected water to ensure that the reactor remains shut down, as well as provide sufficient water depth within the containment sump to ensure adequate net positive suction head (NPSH) for the ECCS [emergency core cooling system] pumps and protect against vortexing. Also, adequate time is provided to ensure the completion of all operator actions necessary for switchover to cold leg recirculation prior to the loss of all usable FWST inventory and loss of suction to the ECCS pumps.

The change in the FWST LOW level setpoint reduces the FWST volume that is delivered to the primary system in the injection phase of a LOCA [loss-of-coolant accident]. Thus, this volume reduction affects the containment pressure response during a LOCA. A reanalysis of the containment pressure response using the NRC-approved methodology of DPC-NE-3004 demonstrates that the peak containment pressure remains below the design limit for the proposed FWST LOW level setpoint.

The LOCA blowdown, refill, and reflood phases of the analysis are not affected by the change in switchover setpoint. Therefore, the fuel clad integrity will not be impacted as a result of this change. The containment response was analyzed and found to be within acceptable limits. Therefore, the fission product barriers are unaffected by this change in setpoint.

The radiological calculations include assumptions regarding the start of ECCS recirculation which could be impacted by