will now comply with applicable NRC requirements.

The Director, Office of Enforcement, may, in writing, relax or rescind any of the above conditions upon demonstration by Mr. Nelson of good cause.

V

In accordance with 10 CFR 2.202, James C. Nelson must, and any other person adversely affected by this Order may, submit an answer to this Order, and may request a hearing on this Order, within 20 days of the date of this Order. Where good cause is shown, consideration will be given to extending the time to request a hearing. A request for extension of time must be made in writing to the Director, Office of Enforcement, U.S. Nuclear Regulatory Commission Washington, D.C. 20555, and include a statement of good cause for the extension. The answer may consent to this Order. Unless the answer consents to this Order, the answer shall, in writing and under oath or affirmation, specifically admit or deny each allegation or charge made in this Order and shall set forth the matters of fact and law on which James C. Nelson or any other person adversely affected relies and the reasons as to why the Order should not have been issued. Any answer or request for a hearing shall be submitted to the Secretary, U.S. Nuclear Regulatory Commission, Attn: Chief, Docketing and Service Section, Washington, DC 20555. Copies also shall be sent to the Director, Office of Enforcement, U.S. Nuclear Regulatory Commission, Washington, DC 20555, to the Assistant General Counsel for Hearings and Enforcement at the same address, to the Regional Administrator, NRC Region II, 101 Marietta Street N.W., Suite 2900, Atlanta, GA 30323, and to James C. Nelson if the answer or hearing request is by a person other than James C. Nelson. If a person other than James C. Nelson requests a hearing, that person shall set forth with particularity the manner in which his or her interest is adversely affected by this Order and shall address the criteria set forth in 10 CFR 2.714(d).

If a hearing is requested by James C. Nelson or any other person whose interest is adversely affected, the Commission will issue an Order designating the time and place of any hearing. If a hearing is held, the issue to be considered at such hearing shall be whether this Order should be sustained.

Pursuant to 10 CFR 2.202(c)(2)(i), James C. Nelson, or any other person adversely affected by this Order, may, in addition to demanding a hearing, at the time the answer is filed or sooner, move the presiding officer to set aside the immediate effectiveness of the Order on the ground that the Order, including the need for immediate effectiveness, is not based on adequate evidence but on mere suspicion, unfounded allegations, or error.

In the absence of any request for hearing, or written approval of an extension of time in which to request a hearing, the provisions specified in Section IV above shall be effective and final 20 days from the date of this Order without further order or proceedings. If an extension of time for requesting a hearing has been approved, the provisions specified in Section IV shall be final when the extension expires if a hearing request has not been received. An answer or a request for hearing shall not stay the immediate effectiveness of this order.

Dated at Rockville, Maryland this 27th day of January 1997.

For the Nuclear Regulatory Commission.

Edward L. Jordan,

Deputy Executive Director for Regulatory Effectiveness, Program Oversight, Investigations and Enforcement.

[FR Doc. 97–2536 Filed 1–31–97; 8:45 am]

BILLING CODE 7590–01–P

[Docket Nos. 50–266 and 50–301]

Wisconsin Electric Power Company (Point Beach Nuclear Plant, Unit Nos. 1 and 2); Exemption

I

Wisconsin Electric Power Company (the licensee) is the holder of Facility Operating License Nos. DRP–24 and DRP–27, which authorize operation of the Point Beach Nuclear Plant, Units 1 and 2, respectively. The licenses provide, among other things, that the licensee is subject to all rules, regulations, and orders of the Commission now or hereafter in effect.

The facility consists of two pressurized-water reactors located at the licensee's site in Manitowoc County, Wisconsin.

II.

In its letter dated July 1, 1996, as supplemented November 18, 1996, the licensee requested an exemption from the Commission's regulations. Title 10 of the *Code of Federal Regulations*, Part 50, Section 60 (10 CFR 50.60), ''Acceptance Criteria for Fracture Prevention Measures for Lightwater Nuclear Power Reactors for Normal Operation,'' states that all lightwater nuclear power reactors must meet the fracture toughness and material surveillance program requirements for the reactor coolant pressure boundary as set forth in Appendices G and H to 10 CFR Part 50. Appendix G to 10 CFR Part 50 defines pressure/temperature (P/T) limits during any condition of normal operation, including anticipated operational occurrences and system hydrostatic tests to which the pressure boundary may be subjected over its service lifetime. It is specified in 10 CFR 50.60(b) that alternatives to the described requirements in Appendices G and H to 10 CFR Part 50 may be used when an exemption is granted by the Commission under 10 CFR 50.12.

To prevent low-temperature overpressure transients that would produce pressure excursions exceeding the P/T limits of Appendix G to 10 CFR Part 50 while the reactor is operating at low temperatures, the licensee installed a low-temperature overpressure protection (LTOP) system. The system includes pressure-relieving devices called power-operated relief valves (PORVs). The PORVs are set at a pressure low enough so that if an LTOP transient occurred, the mitigation system would prevent the pressure in the reactor vessel from exceeding the P/T limits of Appendix G to 10 CFR Part 50. To prevent the PORVs from lifting as a result of normal operating pressure surges (e.g., reactor coolant pumps starting or stopping) with the reactor coolant system in a water solid condition, the operating pressure must be maintained below the PORV setpoint. The maximum LTOP setpoint of 425 psig was approved May 20, 1980, with the issuance of Amendments 45 (DPR–24) and 60 (DPR–27) to the Point Beach operating licenses. This LTOP system received pressure input from the sensing taps located in the reactor coolant system hot leg and at the pressurizer. Subsequent evaluation determined that the methodology used to determine the LTOP system setpoint did not account for the differential pressure across the core during reactor coolant pump operation. A recent Westinghouse calculation (NSAL 93–005) indicated that with both reactor coolant pumps operating, the pressure at core midplane may be as much as 63 psig higher than at the pressure sensing points. To account for this differential pressure, which could cause the reactor vessel midplane pressure to exceed the ASME Section XI, Appendix G limits, the licensee implemented an administrative requirement in 1993 allowing only one reactor coolant pump in operation when reactor coolant temperature is below 160 oF. Plant operation with this restriction places an

unnecessary burden on plant operators to ensure safety limits are maintained.

The licensee has requested the use of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) Case N–514, ''Low Temperature Overpressure Protection,'' which allows exceeding the pressure of the P/T limits of 10 CFR Part 50, Appendix G, by 10 percent. ASME Code Case N–514, the proposed alternate methodology, is consistent with guidelines developed by the ASME Working Group on Operating Plant Criteria to define pressure limits during LTOP events that avoid certain unnecessary operational restrictions, provide adequate margins against failure of the reactor pressure vessel, and reduce the potential for unnecessary activation of pressure-relieving devices used for LTOP. ASME Code Case N–514 has been approved by the ASME Code Committee. The content of this code case has been incorporated into Appendix G of Section XI of the ASME Code and published in the 1993 Addenda to Section XI.

III

Pursuant to 10 CFR 50.12, the Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of 10 CFR Part 50 when (1) the exemptions are authorized by law, will not present an undue risk to public health or safety, and are consistent with the common defense and security and (2) when special circumstances are present. Special circumstances are present whenever, according to 10 CFR 50.12(a)(2)(ii), ''Application of the regulation in the particular circumstances would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule.''

The underlying purpose of 10 CFR 50.60, Appendix G, is to establish fracture toughness requirements for ferritic materials of pressure-retaining components of the reactor coolant pressure boundary to provide adequate margins of safety during any condition of normal operation, including anticipated operational occurrences, to which the pressure boundary may be subjected over its service lifetime. Section IV.A.2 of this appendix requires that the reactor vessel be operated with P/T limits at least as conservative as those obtained by following the methods of analysis and the required margins of safety of Appendix G of the ASME Code, Section XI.

Appendix G of Section XI of the ASME Code requires that the P/T limits be calculated (a) using a safety factor of 2 on the principal membrane (pressure) stresses, (b) assuming a flaw at the surface with a depth of one-quarter (¼) of the vessel wall thickness and a length of 6 times its depth, and (c) using a conservative fracture toughness curve that is based on the lower bound of static, dynamic, and crack arrest fracture toughness tests on material similar to the Point Beach reactor vessel material.

In determining the setpoint for LTOP events, the licensee proposed to use safety margins based on an alternate methodology consistent with the ASME Code Case N–514 guidelines. The ASME Code Case N–514 allows determination of the setpoint for LTOP events such that the maximum pressure in the vessel would not exceed 110 percent of the P/T limits of the existing ASME Code, Section XI, Appendix G. This approach results in a safety factor of 1.8 on pressure. All other factors, including assumed flaw size and fracture toughness, remain the same. Although this methodology would reduce the safety factor on pressure, it was demonstrated in the Bases of the ASME Code Case N–514 that due to the isothermal nature of LTOP events, the margin with respect to toughness for LTOP transients is within the range provided by ASME, Section XI, Appendix G for normal heatup and cooldown in the low temperature range. Thus, applying Code Case N–514 will satisfy the underlying purpose of 10 CFR 50.60 for fracture toughness requirements. Further, by relieving the operational restrictions, the potential for undesirable lifting of the PORV would be reduced, thereby improving plant safety.

IV

For the foregoing reasons, the NRC staff has concluded that the licensee's proposed use of the alternate methodology in determining the acceptable setpoint for LTOP events will not present an undue risk to public health and safety and is consistent with the common defense and security. The NRC staff has determined that there are special circumstances present, as specified in 10 CFR 50.12(a)(2)(ii), in that application of 10 CFR 50.60 is not necessary in order to achieve the underlying purpose of this regulation.

Accordingly, the Commission has determined that, pursuant to 10 CFR 50.12(a), an exemption is authorized by law, will not endanger life or property or common defense and security, and is otherwise in the public interest. Therefore, the Commission hereby grants an exemption from the requirements of 10 CFR 50.60 such that in determining the setpoint for LTOP events, the Appendix G curves for P/T limits are not exceeded by more than 10 percent. This exemption is applicable only to LTOP conditions during normal operation.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will not have a significant effect on the quality of the human environment (61 FR 66062).

This exemption is effective upon issuance.

For the Nuclear Regulatory Commission.

Dated at Rockville, Maryland, this 27th day of January 1997.

Frank J. Miraglia

Acting Director, Office of Nuclear Reactor Regulation.

[FR Doc. 97–2537 Filed 1–31–97; 8:45 am]

BILLING CODE 7590–01–P

OFFICE OF PERSONNEL MANAGEMENT

Federal Prevailing Rate Advisory Committee; Open Committee Meeting

According to the provisions of section 10 of the Federal Advisory Committee Act (Pub. L. 92–463), notice is hereby given that meetings of the Federal Prevailing Rate Advisory Committee will be held on Wednesday, February 5, 1997.

The meeting will start at 10:45 a.m. and will be held in Room 5A06A, Office of Personnel Management Building, 1900 E Street, NW., Washington, DC.

The Federal Prevailing Rate Advisory Committee is composed of a Chair, five representatives from labor unions holding exclusive bargaining rights for Federal blue-collar employees, and five representatives from Federal agencies. Entitlement to membership on the Committee is provided for in 5 U.S.C. 5347.

The Committee's primary responsibility is to review the Prevailing Rate System and other matters pertinent to establishing prevailing rates under subchapter IV, chapter 53, 5 U.S.C., as amended, and from time to time advise the Office of Personnel Management.

These scheduled meetings will start in open session with both labor and management representatives attending. During the meeting either the labor members or the management members may caucus separately with the Chair to devise strategy and formulate positions. Premature disclosure of the matters discussed in these caucuses would unacceptably impair the ability of the Committee to reach a consensus on the matters being considered and would disrupt substantially the disposition of