

**NATIONAL SCIENCE FOUNDATION****United States Antarctic Program (USAP) Blue Ribbon Panel; Notice of Meeting**

In accordance with the Federal Advisory Committee Act (Pub. L. 92-463, as amended), the National Science Foundation announces the following meeting.

*Name:* United States Antarctic (USAP) Program Blue Ribbon Panel (#1531).

*Date & Time:* December 20, 8:00 a.m.–6:00 p.m.; December 31, 1996, 8:30 a.m.–5 p.m.

*Place:* Room 1235, NSF, 4201 Wilson Boulevard, Arlington, VA.

*Type of Meeting:* Open.

*Contact Person:* Guy G. Guthridge, Office of Polar Programs, Room 755, National Science Foundation, 4201 Wilson Boulevard, Arlington, Virginia 22230. Telephone: (703) 306-1031.

*Minutes:* May be obtained from the contact person listed above.

*Purpose of Meeting:* Examine a full range of infrastructure, management, and scientific options for the United States Antarctic Program so that the Foundation will be able to maintain the high quality of research and implement U.S. policy in Antarctica under realistic budget scenarios.

*Agenda:* The committee will continue analysis begun at its first meeting (October 11-12, 1996). It will receive presentations from Antarctic experts and will discuss options in the areas of research, research support, contractor tasking, military transition, cost-saving initiatives, health and safety context, environment and waste management, South Pole redevelopment, international aspects, science users' perspectives, and interagency involvement.

Dated: November 27, 1996.

M. Rebecca Winkler,

*Committee Management Officer.*

[FR Doc. 96-30777 Filed 12-3-96; 8:45 am]

**BILLING CODE 7555-01-M**

**NATIONAL TRANSPORTATION SAFETY BOARD****Sunshine Act Meeting**

**TIME:** 9:30 a.m., Wednesday, December 11, 1996.

**PLACE:** The Board Room, 5th Floor, 490 L'Enfant Plaza, S.W., Washington, D.C. 20594.

**STATUS:** Open.

**MATTERS TO BE DISCUSSED:**

- 6781 Aviation Accident Report:  
Ground Spoiler Activation in Flight/  
Hard Landing, ValuJet Airlines Flight  
558, Douglas DC-9-32, N922VV,  
Nashville International Airport,  
Nashville, Tennessee, January 7, 1996.
- 6675A Railroad Accident Report:  
Derailment of Atchison, Topeka and

Santa Fe Railway Company Train H-BALT1-31, Near Cajon Junction,  
California, February 1, 1996.

**NEWS MEDIA CONTACT:** Telephone: (202) 314-6100.

**FOR MORE INFORMATION CONTACT:** Bea Hardesty, (202) 314-6065.

November 29, 1996.

Bea Hardesty,

*Federal Register Liaison Officer.*

[FR Doc. 96-30937 Filed 12-2-96; 11:36 am]

**BILLING CODE 7533-01-P**

**NUCLEAR REGULATORY COMMISSION**

[Docket No. 50-368]

**Arkansas Nuclear One, Unit 2; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination and Opportunity for a Hearing**

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-6 issued to Entergy Operations, Inc. for operation of Arkansas Nuclear One, Unit 2 (ANO-2) located in Pope County, Arkansas.

The proposed amendments would change the surveillance requirements for the Arkansas Nuclear One, Unit 2 (ANO-2) steam generator tubing. This proposed change references a new generic topical report (CEN-630-P, "Repair of 3/4" O.D. Steam Generator Tubes Using Leak-Tight Sleeves," Revision 01, November 1996).

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

**Criterion 1—Does Not Involve a Significant Increase in the Probability or Consequences of an Accident Previously Evaluated.**

The proposed amendment continues to allow the ABB/Combustion Engineering (CE) tungsten inert gas (TIG) welded expansion transition zone (ETZ) and tube support sleeves to be used as an alternate tube repair method for the Arkansas Nuclear One, Unit 2 (ANO-2) steam generators along with process improvements which are included in the topical report to be referenced. The sleeve configuration was designed and analyzed in accordance with the criteria of Regulatory Guide (RG) 1.121 and Section III of the ASME Code and is unaffected by the enhancements that will be implemented. The consequences of leakage through the sleeved region of the tube, including the proposed enhancements, is bounded by the existing steam generator tube rupture (SGTR) analysis included in the ANO-2 Safety Analysis Report.

The proposed change reflects enhancements made to the installation inspection process which is identified in the currently licensed topical report (CEN-601-P, Revision 01-P). The new topical report (CEN-630-P, Revision 01) specifies that proper cleaning and inspection of the weld zone be performed prior to sleeve installation. Also, eddy current testing (ECT) has been added as part of the sleeve acceptance criteria to ensure the structural integrity of the tube-to-sleeve weld joint. The ECT added allows disposition of certain non-significant indications outside the sleeve's pressure boundary without subsequent repair of the tube. Other changes caused by referencing a generic topical report, instead of a site-specific analysis, increase the conservatism already present with the currently licensed process. The lower primary-to-secondary leakage limit ensures that any dose contributed from a potential steam generator tube leak will be considerably lower than the dosage specified in 10 CFR 100.

Therefore, this change does not involve a significant increase in the probability or consequences of any accident previously evaluated.

**Criterion 2—Does Not Create the Possibility of a New or Different Kind of Accident from any Previously Evaluated.**

The proposed change to implement CEN-630-P, Revision 1, will not create a new or different type of accident. The changes reflect enhancements to the currently licensed installation/inspection process and would not affect any hypothetical accident as a result of potential tube or sleeve degradation in the repaired portion of the tube. Such hypothetical accidents remain bounded by the existing SGTR analysis. The sleeve design does not affect any other component or portion of the steam generator tube outside of the immediate area repaired.

Therefore, this change does not create the possibility of a new or different kind of accident from any previously evaluated.

**Criterion 3—Does Not Involve a Significant Reduction in the Margin of Safety.**

The currently licensed TIG welded sleeving repair of degraded steam generator tubes has been shown by analysis to restore

the integrity of the tube to its original design basis condition. By implementing the proposed enhancements, the quality of the sleeve welds will be increased thereby reducing the potential for leaving a weld indication in service.

Installation/inspection enhancements are being made to a process which is currently licensed for use at ANO-2 by the NRC staff. These enhancements would not have any adverse effects on the previously evaluated design transient or accident analysis. The enhancements only specify inspection methods of the weld zones which will ensure the integrity of the pressure boundary.

Reducing the allowable primary-to-secondary leakage rate through the steam generators actually increases the margin of safety by reducing potential dose contribution due to steam generator tube leakage.

Therefore, this change does not involve a significant reduction in the margin of safety.

Therefore, based upon the reasoning presented above and the previous discussion of the amendment request, Entergy Operations has determined that the requested change does not involve a significant hazards consideration.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the Federal Register a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules Review and Directives Branch, Division of Freedom of Information and Publications

Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this Federal Register notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By January 3, 1997, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, N.W., Washington, DC, and at the local public document room located at the Tomlinson Library, Arkansas Tech University, Russellville, AR 72801. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should

also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to William D. Beckner, Director, Project Directorate IV-1: Petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this Federal Register notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to Mr. Nicholas S. Reynolds, Winston & Strawn, 1400 L Street, N.W., Washington, D.C. 20005-3502, attorney for the licensee.

Non-timely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1) (i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated November 24, 1996, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Tomlinson Library, Arkansas Tech University, Russellville, Arkansas 72801.

Dated at Rockville, Maryland, this 29th day of November 1996.

For the Nuclear Regulatory Commission,  
Kombiz Salehi,

*Acting Project Manager, Project Directorate IV-1, Division of Reactor Projects III/IV, Office of Nuclear Reactor Regulation.*

[FR Doc. 96-30900 Filed 12-3-96; 8:45 am]

BILLING CODE 7590-01-P

[Docket No. 72-17 (50-344)]

**Portland General Electric Company, et al.; Notice of Issuance of Environmental Assessment and Finding of No Significant Impact for the Independent Spent Fuel Storage Installation at Trojan Nuclear Plant**

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of a materials license under the requirements of Title 10, Code of Federal Regulations (10 CFR), Part 72, to Portland General Electric Company, et al. (PGE or the applicant), authorizing receipt and storage of spent fuel in an independent spent fuel storage installation (ISFSI) located at its Trojan Nuclear Plant (TNP) in Columbia County, Oregon. The Commission's Office of Nuclear Material Safety and Safeguards, Spent Fuel Project Office, has completed its environmental review in support of the issuance of a materials license. The "Environmental Assessment (EA) Related to the Construction and Operation of the Trojan Independent Spent Fuel Storage Installation" has been issued in accordance with 10 CFR Part 51.

**Summary of Environmental Assessment**

*Description of the Proposed Action:* The proposed licensing action would authorize the applicant to construct and operate a dry storage ISFSI at the Trojan site. The primary function of the ISFSI is to provide interim storage of spent fuel assemblies, fuel debris, and greater than Class C (GTCC) waste, which were generated at the Trojan Nuclear Plant during its operation.<sup>1</sup>

Currently, the spent fuel and fuel debris are stored in the Trojan spent fuel pool.

PGE has selected a dry storage system using Sierra Nuclear Corporation's TranStor Storage System design. The TranStor Storage System is a vertical, dry storage system which utilizes a ventilated concrete storage cask and a seal-welded steel basket to store spent nuclear fuel assemblies, fuel debris and GTCC waste.

<sup>1</sup> At present, licenses issued under the Commission's regulations at 10 CFR Part 72 are limited to the storage of spent fuel and other radioactive materials associated with spent fuel storage in an ISFSI. Storage of GTCC waste is not within the scope of a Part 72 license. However, on November 2, 1995, PGE submitted a petition for rulemaking requesting that the Commission amend its Part 72 regulations to specifically provide for the storage of GTCC waste in an ISFSI. See 61 FR 3619 (1996). Consideration of the inclusion of this type of waste in the EA for the Trojan ISFSI should obviate the necessity for revisiting the environmental impacts of storage of GTCC waste at Trojan if the Commission grants PGE's petition and amends its regulations as requested.

The license for an ISFSI under 10 CFR Part 72 is issued for 20 years. However, the licensee may apply to the Commission to renew the license, if necessary, prior to its expiration.

*Need for the Proposed Action:* TNP was shutdown in November 1992, and on January 27, 1993, PGE notified the NRC of its decision to permanently cease power operation and subsequently defueled the reactor, storing the spent fuel in the TNP spent fuel pool. Currently, PGE has a possession-only license under 10 CFR Part 50 and applied to terminate its license on January 25, 1995, by submitting a decommissioning plan. The licensee proposed to decommission the facility using a dismantlement or DECON approach as defined in the "Final Generic Environmental Impact Statement on Decommissioning of Nuclear Facilities," NUREG-0586, dated August 1988.

PGE's plans for decommissioning the TNP include decontamination and dismantlement of contaminated structures, systems, and components. To facilitate decommissioning, the spent fuel and other contents of the spent fuel pool must be relocated. The licensee determined that an ISFSI would be the most economical method for the temporary storage of the spent fuel until acceptance of the spent fuel by the U.S. Department of Energy, which is responsible for the permanent disposal of spent fuel. Relocating the spent fuel to an ISFSI would allow TNP to proceed with decontamination and dismantlement of the structures, systems, and components without impacting the safe storage of spent fuel.

*Environmental Impacts of the Proposed Action:* As discussed in the EA, no significant construction impacts are anticipated. Trojan ISFSI construction activities will affect only a small fraction of the land area of TNP. With good construction practices, the potential for fugitive dust, erosion, and noise, typical of the planned construction activities, can be controlled to insignificant levels. The only resources irretrievably committed are the steel, concrete, and other construction materials used in the ISFSI pad, storage casks, and any operating equipment.

As discussed in the EA, there will be no radiological liquid or gaseous effluents during normal operation of the ISFSI. The estimated doses to both occupational workers and members of the public are below regulatory limits.

As discussed in the EA, no significant radiological impacts are expected during operation of the ISFSI. The only environmental interface of the ISFSI is