

the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If the amendment is issued before the expiration of the 30-day hearing period, the Commission will make a final determination on the issue of no significant hazards consideration. If a hearing is requested, the final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1 (800) 248-5100 (in Missouri 1 (800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to Herbert N. Berkow: petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this Federal Register notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to M. Stanford Blanton, Esq., Balch and Bingham, P.O.

Box 306, 1710 Sixth Avenue, Birmingham, Alabama, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated June 24, 1996, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room, located at the Houston-Love Memorial Library, 212 W. Burdeshaw Street, P.O. Box 1369, Dothan, Alabama.

Dated at Rockville, Maryland, this 27th day of June 1996.

For the Nuclear Regulatory Commission.
Byron L. Siegel,

Project Manager, Project Directorate II-2, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

[FR Doc. 96-16964 Filed 7-2-96; 8:45 am]

BILLING CODE 7590-01-P

Boraflex Degradation in Spent Fuel Pool Storage Racks; Issued

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of Issuance.

SUMMARY: The Nuclear Regulatory Commission (NRC) has issued Generic Letter 96-04 to notify all licensees of nuclear power reactors about problems that have been encountered with using Boraflex in spent fuel storage racks for the nonproductive absorption of neutrons, and for licensees that use Boraflex, to request implementation of certain actions and require the submittal of a written response. This generic letter is available in the NRC Public Document Room under accession number 9606240132.

DATES: The generic letter was issued on June 26, 1996.

ADDRESSEES: Not applicable.

FOR FURTHER INFORMATION CONTACT: Laurence I. Kopp at (301) 415-2879.

SUPPLEMENTARY INFORMATION: The information that is being requested will enable the NRC staff to determine whether licensees are complying with the current licensing basis for the facility with respect to GDC 62 for the

prevention of criticality in fuel storage and handling, and 5-percent subcriticality margins that are either contained in the technical specifications, or committed to in the updated FSARs, of plants containing Boraflex in the spent fuel storage racks. The staff is not establishing a new position for such compliance in this generic letter.

Dated at Rockville, Maryland, this 26th day of June, 1996.

For the Nuclear Regulatory Commission.
Elinor G. Adensam,
Deputy Director, Division of Reactor Program Management, Office of Nuclear Reactor Regulation.

[FR Doc. 96-16963 Filed 7-2-96; 8:45 am]

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Advisory Committee on Reactor Safeguards; Meeting Notice

In accordance with the purposes of Sections 29 and 182b. of the Atomic Energy Act (42 U.S.C. 2039, 2232b), the Advisory Committee on Reactor Safeguards will hold a meeting on August 8-10, 1996, in Conference Room T-2B3, 11545 Rockville Pike, Rockville, Maryland. The date of this meeting was previously published in the Federal Register on Monday, November 27, 1995 (60 FR 58393).

Thursday, August 8, 1996

8:30 A.M.-8:45 A.M.: Opening Remarks by the ACRS Chairman

(Open)—The ACRS Chairman will make opening remarks regarding conduct on the meeting and comment briefly regarding items of current interest. During this session, the Committee will discuss priorities for preparation of ACRS reports.

8:45 A.M.-10:45 A.M.: Supplemental Safety Evaluation Report for Evolutionary Plant Designs

(Open/Closed)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff, General Electric Nuclear Energy (GENE), and ABB-Combustion Engineering (ABB-CE) regarding the proposed changes to the GENE Advanced Boiling Water Reactor (ABWR) and ABB-CE System 80+ evolutionary plant designs and the associated NRC staff Safety Evaluation Report. Other interested parties will participate, as appropriate.

A portion of this session may be closed to discuss GENE and ABB-CE proprietary information applicable to this matter.