

them by the Enforcement Conference. The day prior to that Conference, the Licensee submitted a lengthy letter addressing the violations and the status of corrective actions. The information in this letter was not completely accurate and at the Conference several corrections were requested. These corrections were later submitted by the Licensee. In addition, the NRC staff had questioned the RSO's ability to meet his responsibilities for the numerous facilities and Licensee management had indicated that it intended to request a separate license for a New Jersey facility in order to relieve the RSO of some responsibilities, but it had not yet done so. In addition, the Licensee did not consider the need to apply similar corrective actions at the other facilities covered by the license.

Although the Licensee had recognized that it had weaknesses in its program and had engaged a consultant to assist the RSO, and these actions led to eventual good comprehensive corrective action, they were not sufficiently prompt and comprehensive as of the time of the Enforcement Conference to provide a basis for mitigating the civil penalty.

3. NRC Conclusion

The NRC has concluded that the violations occurred as stated and an adequate basis for mitigation of the civil penalty was not provided by the licensee. Consequently, the proposed civil penalty in the amount of \$2,500 should be imposed.

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[Docket Nos. 50-413 and 50-414]

Duke Power Company, et al., Catawba Nuclear Station, Units 1 and 2; Correction to Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission published a Notice of Consideration of Issuance of Amendments in the Federal Register (60 FR 58109 dated November 24, 1995), to Duke Power Company, et al., for the Catawba Nuclear Station, Units 1 and 2. Correction is being made on page 58110, third column, last paragraph, first sentence; the 30-day notice period ending date should read "By December 26, 1995, * * *" instead of "By December 18, 1995, * * *"

Dated at Rockville, Maryland, this 28th day of November 1995.

For the Nuclear Regulatory Commission,
Robert E. Martin,
Senior Project Manager, Project Directorate II-2, Division of Reactor Projects—III, Office of Nuclear Reactor Regulation.

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[Docket Nos. 50-277 and 50-278]

Peco Energy Company; Notice of Consideration of Issuance of Amendments to Facility Operating License, Proposed no Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. DPR-44 and DPR-56 issued to the PECO Energy Company (the licensee) for operation of the Peach Bottom Atomic Power Station, Units 2 and 3, located in York County, Pennsylvania.

The proposed amendments would revise surveillance requirements for the high pressure coolant injection and reactor core isolation cooling systems and would make an administrative change to Section 5.5.7 of the technical specifications to eliminate reference to a section which was previously eliminated.

Before issuance of the proposed license amendments, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

(1) The proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated because the changes will not alter assumptions relative to initiation and mitigation of analyzed events. These changes will not alter the operation of process variables, or SSC [system, structure or component] as described in the safety analysis. These changes do not involve any physical changes to plant SSC or the manner in which these SSC are operated, maintained, modified or inspected. Routine testing is not assumed to be an initiator of any analyzed event. The proposed changes will not alter the operation of equipment assumed to be available for the mitigation of accidents or transients by the plant safety analysis or licensing basis. These changes have been

confirmed to ensure no previously evaluated accident has been adversely affected. The proposed lower test pressure for the HPCI [high pressure coolant injection] and RCIC [reactor core isolation cooling] system flow testing is consistent with the minimum EHC [electro-hydraulic control] pressure setpoint at which reactor power can be increased without the need to adjust the EHC pressure setpoint during operation in MODE 1. Increasing the lower test pressure from 920 psig to 940 psig does not impact when the performance of the test is required. The proposed upper test pressure for the HPCI and RCIC system flow testing is consistent with the Reactor Steam Dome Pressure Limit in Specification 3.4.10. Additionally, the HPCI and RCIC systems are both designed to provide adequate core cooling at reactor pressures from 150 psig to 1150 psig. SR [surveillance requirement] 3.5.1.8 and SR 3.5.3.3 still will require verifying HPCI and RCIC pumps can develop the required flow rates against system head corresponding to reactor pressure. Therefore, the proposed changes provide adequate assurance that the HPCI and RCIC systems will be maintained operable. In addition, these proposed changes eliminate the need to adjust reactor pressure from normally stable plant conditions to perform the test. As such, the probability of plant transients is expected to be reduced. Therefore, the proposed changes will not involve a significant increase in the probability or consequences of an accident previously evaluated.

(2) The proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated because the proposed changes do not alter the plant configuration (no new or different type of equipment will be installed or removed) and will not alter the method used by any system to perform its design function. The proposed changes do not allow plant operation in any mode that is not already evaluated in the SAR [safety analysis report]. Therefore, these changes will not create the possibility of a new or different kind of accident from any accident previously evaluated.

(3) The proposed changes do not involve a significant reduction in a margin of safety. The proposed change to the VFTP [ventilation filter test program] in Section 5.5.7 is administrative in nature and does not involve any technical changes. This proposed change will not reduce a margin of safety because it has no impact on any safety analysis assumptions. Because this change is administrative in nature, no question of safety is involved. The proposed changes also revise the upper and lower test pressure for the HPCI and RCIC system high pressure flow tests. These changes do not impact safety analysis assumptions or the ability of the HPCI and RCIC systems to perform their design functions. The HPCI and RCIC systems are designed to provide adequate core cooling at reactor pressures from 150 psig to 1150 psig. SR 3.5.1.8 and SR 3.5.3.3 still will require verifying HPCI and RCIC pumps can develop the required flow rates against system head corresponding to reactor pressure. The proposed lower test pressure for the HPCI and RCIC system flow testing is