

granting this exemption will not have a significant impact on the human environment (60 FR 42189).

This exemption is effective upon issuance and shall expire at the completion of the 3R6 refueling outage.

Dated at Rockville, Maryland, this 16th day of August 1995.

For the Nuclear Regulatory Commission.

**Jack W. Roe,**

*Director, Division of Reactor Projects III/IV, Office of Nuclear Reactor Regulation.*

[FR Doc. 95-20749 Filed 8-21-95; 8:45 am]

BILLING CODE 7590-01-P

**[Docket No. 50-400]**

**Carolina Power & Light Company; Notice of Withdrawal of Application for Amendment to Facility Operating License**

The U.S. Nuclear Regulatory Commission (the Commission) has granted the request of Carolina Power & Light Company (the licensee) to withdraw its March 20, 1995 application for proposed amendment to Facility Operating License No. NPF-63 for the Shearon Harris Nuclear Power Plant, Unit No. 1, located in New Hill, North Carolina 27562.

The proposed amendment would have revised the technical specifications to allow the relocation of cycle-specific Overpower and Overtemperature Delta T trip setpoint parameters to the Core Operating Limits Report. The Commission had previously issued a Notice of Consideration of Issuance of Amendment published in the **Federal Register** on April 26, 1995 (60 FR 20515). However, by letter dated August 3, 1995, the licensee withdrew the proposed change.

For further details with respect to this action, see the application for amendment dated March 20, 1995, and the licensee's letter dated August 3, 1995, which withdrew the application for license amendment. The above documents are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Cameron Village Regional Library, 1930 Clark Avenue, Raleigh, North Carolina 27605.

Dated at Rockville, Maryland, this 16th day of August 1995.

For the Nuclear Regulatory Commission.

**Ngoc B. Le,**

*Project Manager, Project Directorate II-1, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.*

[FR Doc. 95-20744 Filed 8-21-95; 8:45 am]

BILLING CODE 7590-01-M

**[Docket Nos. 50-277 and 50-278]**

**PECO Energy Company; Notice of Issuance of Amendment to Facility Operating License**

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment Nos. 209 and 213 to Facility Operating Licenses Nos. DPR-44 and DPR-56 issued to PECO Energy Company (the licensee), which revised the Technical Specifications for operation of the Peach Bottom Atomic Power Station, Units 2 and 3, located in York County, Pennsylvania. The amendment is effective as of the date of issuance.

The amendment modified the Technical Specifications to provide for an increased allowed out-of-service time for the Peach Bottom emergency diesel generators based on the availability of a power tie-line from the Conowingo Hydroelectric Station.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment to Facility Operating License and Opportunity for a Hearing in connection with this action was published in the **Federal Register** on June 7, 1995 (60 FR 30120). No request for a hearing or petition for leave to intervene was filed following this notice.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of the amendment will not have a significant effect on the quality of the human environment (60 FR 40866).

For further details with respect to the action see (1) the application for amendment dated April 7, 1994 and supplemented by letters dated June 2 and September 6, 1994 and June 16 and July 13, 1995, (2) Amendment Nos. 209/213 to Licenses Nos. DPR-44 and DPR-56, (3) the Commission's related Safety Evaluation, and (4) the Commission's Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street NW., Washington, DC, and at the

local public document room located at Government Publications Section, State Library of Pennsylvania, (REGIONAL DEPOSITORY) Education Building, Walnut Street and Commonwealth Avenue, Box 1601, Harrisburg, Pennsylvania.

Dated at Rockville, Maryland, this 16th day of August 1995.

For the Nuclear Regulatory Commission.

**Joseph W. Shea,**

*Project Manager, Project Directorate I-2, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.*

[FR Doc. 95-20743 Filed 8-21-95; 8:45 am]

BILLING CODE 7590-01-P

**[Docket Nos. 50-352 and 50-353]**

**Pennsylvania Power and Light Company (Susquehanna Steam Electric Station, Units 1 and 2); Exemption**

**I**

Pennsylvania Power and Light Company (the licensee), is the holder of Facility Operating License Nos. NPF-14 and NPF-22, which authorize operation of the Susquehanna Steam Electric Station (SSES), Units 1 and 2. The licenses provide, among other things, that the licensee is subject to all rules, regulations, and orders of the Nuclear Regulatory Commission (the Commission) now and hereafter in effect. The facilities consist of two boiling water reactors located in Luzerne County, Pennsylvania.

**II**

Section 50.54(o) of 10 CFR Part 50 requires that primary reactor containments for water cooled power reactors be subject to the requirements of Appendix J to 10 CFR Part 50. Appendix J contains the leakage test requirements, schedules, and acceptance criteria for tests of the leak tight integrity of the primary reactor containment and systems and components which penetrate the containment. Sections II.H.4 and III.C.2(a) of Appendix J to 10 CFR Part 50 require leak rate testing of Main Steam Isolation Valves (MSIVs) at the calculated peak containment pressure related to the design basis accident, and Section III.C.3 requires that the measured leak rates be included in the combined local leak rate test results. The proposed deletion of the MSIV Leakage Control System (LCS), and proposed use of an alternate leakage pathway affects the description of an existing exemption which allows the leak rate testing of the MSIVs at a reduced pressure and the exclusion of