

and thus to ensure the reliability of, the OCNCS reactor vessel internal components.

Based on the above, the staff has concluded that suspension of the Oyster Creek Nuclear Generating Station operating license due to embrittlement and cracking of the reactor vessel internal components is not warranted. As stated previously, continued monitoring of reactor vessel internals as required by the ASME Code and the licensee's inservice inspection program will provide assurance that degradation of components will be detected and appropriate action will be taken.

*B. Petitioners request that the NRC suspend the OCNCS operating license until the Licensee provides an analysis regarding the synergistic effects of through-wall cracking of multiple safety-class components.* The majority of reactor internals are fabricated from high-toughness materials such as stainless steel and were designed with significant margins on allowable stresses. As such, cracking must be severe to adversely impact plant safety. It is unlikely that licensee inspections would not find such severe degradation. In fact, identification and sizing of the cracks in the H4 location on the OCNCS core shroud are good examples of the effectiveness of the inspections. In addition, NRC staff evaluation of the results from internals inspections performed to date at OCNCS resulted in the conclusion that ASME Code safety margins have been maintained.

The Licensee has not provided an analysis to NRC that addresses the synergistic effects of cracking in multiple safety-class components. The NRC staff does not consider the lack of such an analysis to be a safety concern because of the inspection requirements that pertain to reactor internals and the results of inspections performed to date. See Section III.A, *supra*.

Continued monitoring of reactor vessel internals as required by the ASME Code and the licensee's inservice inspection program will provide information about the structural integrity of reactor vessel internals in the long term. The NRC has asked the BWR Vessel Internals Project (BWRVIP), an industry group, to develop an assessment to address cracking in BWR reactor vessel internals. A report from the BWRVIP is expected on the long term effects of reactor vessel internals cracking in late 1995. In addition, the NRC has undertaken a longer term evaluation of the effects of cracking in multiple reactor vessel internal components that will be approached with appropriate treatment of the key variables (safety function, material

susceptibility, loading, environment, etc.).

Based on the above, the staff has concluded that suspension of the Oyster Creek Nuclear Generating Station license, due to the lack of an analysis of the synergistic effects of through-wall cracking of safety-class reactor internal components, is not warranted.

#### IV. Conclusion

The Petitioners requested that the NRC suspend the operating license of Oyster Creek Nuclear Generating Station until: (1) the Licensee inspects, repairs, or replaces, all safety-class reactor internal components subject to embrittlement and cracking, and (2) the Licensee provides an analysis regarding the synergistic effects of through-wall cracking of multiple safety-class components. For the reasons discussed above, I conclude that the issues raised by the Petitioners are being adequately addressed and that there is no basis for suspending the OCNCS operating license or taking the other requested action. Accordingly, the Petitioners' above-referenced requests are denied.

A copy of this Partial Director's Decision will be filed with the Secretary of the Commission for review as stated in 10 CFR 2.206(c). This Decision will become the final action of the Commission 25 days after issuance unless the Commission, on its own motion, institutes review of the Decision within that time.

Dated at Rockville, Maryland, this 4th day of August 1995.

For the Nuclear Regulatory Commission.

**William T. Russell,**

*Director, Office of Nuclear Reactor Regulation.*

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[Docket Nos. 50-280 and 50-281]

#### Virginia Electric and Power Co.; Issuance of Amendment to Facility Operating License

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment Nos. 203 and 203 to Facility Operating License Nos. DPR-32 and DPR-37 issued to Virginia Electric and Power Company, which revised the License and the Technical Specifications for operation of the Surry Power Station, Unit Nos. 1 and 2 located in Surry County, Virginia. The amendments are effective as of the date of issuance.

The amendments modified the Licenses and the Technical Specifications to increase the authorized

core power level for Surry, Units 1 and 2, from 2441 MWt to 2546 MWt.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity for Hearing in connection with this action was published in the **Federal Register** on December 16, 1994 (59 FR 65085). No request for a hearing or petition for leave to intervene was filed following this notice.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement.

Based upon the environmental assessment, the Commission has concluded that the issuance of the amendment will not have a significant effect on the quality of the human environment (60 FR 32356).

For further details with respect to the action see (1) the application for amendment dated August 30, 1994, and supplemented February 6, February 13, February 27, March 23, March 28, April 13, April 20, April 28, May 5, and June 8, 1995, (2) Amendment Nos. 203 and 203 to License Nos. DPR-32 and DPR-37, (3) the Commission's related Safety Evaluation, and (4) the Commission's Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street NW., Washington, DC, and at the local public document room located at the Swen Library, College of William and Mary, Williamsburg, Virginia 23185.

Dated at Rockville, Maryland, this 3rd day of August 1995.

For The Nuclear Regulatory Commission.

**Bart C. Buckley,**

*Senior Project Manager, Project Directorate II-1, Division of Reactor Projects, Office of Nuclear Reactor Regulation.*

[FR Doc. 95-19767 Filed 8-9-95; 8:45 am]

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#### SECURITIES AND EXCHANGE COMMISSION

#### Requests Under Review by Office of Management and Budget

Agency Clearance Officer: Michael E. Bartell (202) 942-8800