

Time: 8:30 a.m. to 5:00 p.m.

Room: 430.

Program: This meeting will review Biennial/Triennial applications submitted by state humanities councils to Federal-State Partnership, for projects beginning after November, 1995.

19. Date: May 22, 1995.

Time: 9:00 a.m. to 5:30 p.m.

Room: 315.

Program: This meeting will review proposals submitted to the April 1, 1995 deadline in the Higher Education Program, for projects beginning after October, 1995.

20. Date: May 24, 1995.

Time: 9:00 a.m. to 5:30 p.m.

Room: 415.

Program: This meeting will review proposals submitted to the April 1, 1995 deadline in the Higher Education Program, for projects beginning after October, 1995.

21. Date: May 25-26, 1995.

Time: 9:00 a.m. to 5:30 p.m.

Room: 430.

Program: This meeting will review applications for Special Projects of the Special Competition deadline for April 28, 1995, submitted to the Division of Public Programs, for projects beginning after September, 1995.

David C. Fisher,

Advisory Committee, Management Officer.

[FR Doc. 95-9678 Filed 4-18-95; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-306]

Northern States Power Company (Prairie Island Unit 2); Exemption

I

Northern States Power Company (NSP, the licensee) is the holder of Facility Operating License No. DPR-60 which authorizes operation of Prairie Island Nuclear Generating Plant, Unit No. 2. The unit is a pressurized water reactor (PWR) located in Goodhue County, Minnesota. The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the Nuclear Regulatory Commission (the Commission) now or hereafter in effect.

II

Pursuant to 10 CFR 50.12(a), the NRC may grant exemptions from the requirements of the regulations (1) which are authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security; and (2) where special circumstances are present.

Section III.D.1.(a) of Appendix J to 10 CFR Part 50 requires the performance of

three Type A containment integrated leakage rate tests (ILRTs), at approximately equal intervals during each 10-year service period of the primary containment. The third test of each set shall be conducted when the plant is shut down for the 10-year inservice inspection of the primary containment.

III

By letters dated February 23 and March 3, 1995, NSP requested temporary relief from the requirement to perform a set of three Type A tests at approximately equal intervals during each 10-year service period of the primary containment. The requested exemption would permit a one-time interval extension of the third Type A test by approximately 24 months (from the 1995 refueling outage, currently scheduled to begin in May 1995, to the 1997 refueling outage) and would permit the third Type A test of the second 10-year inservice inspection period to not correspond with the end of the current American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) inservice inspection interval.

The licensee's request cites the special circumstances of 10 CFR 50.12, paragraph (a)(2)(ii), as the basis for the exemption. NSP points out that the existing Type B and C testing programs are not being modified by this request and will continue to effectively detect containment leakage caused by the degradation of active containment isolation components as well as containment penetrations. It has been the consistent and uniform experience at Prairie Island Nuclear Generating Plant, Unit No. 2, during the five Type A tests conducted from 1977 to date, that any significant containment leakage paths are detected by the Type B and C testing. The Type A test results have only been confirmatory of the results of the Type B and C test results.

IV

Section III.D.1.(a) of Appendix J to 10 CFR Part 50 states that a set of three Type A leakage rate tests shall be performed at approximately equal intervals during each 10-year service period.

The licensee proposes an exemption to this section which would provide a one-time interval extension for the Type A test by approximately 24 months. The Commission has determined, for the reasons discussed below, that pursuant to 10 CFR 50.12(a)(1) this exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with the

common defense and security. The Commission further determines that special circumstances, as provided in 10 CFR 50.12(a)(2)(ii), are present justifying the exemption; namely, that application of the regulation in the particular circumstances is not necessary to achieve the underlying purpose of the rule.

The underlying purpose of the requirement to perform Type A containment leak rate tests at intervals during the 10-year service period is to ensure that any potential leakage pathways through the containment boundary are identified within a time span that prevents significant degradation from continuing or becoming unknown. The NRC staff has reviewed the basis and supporting information provided by the licensee in the exemption request. The NRC staff has noted that the licensee has a good record of ensuring a leak-tight containment. All Type A tests have passed with significant margin and the licensee has noted that the results of the Type A testing have been confirmatory of the Type B and C tests which will continue to be performed. The licensee has stated that it will perform the general containment inspection although it is only required by Appendix J (Section V.A.) to be performed in conjunction with Type A tests. The NRC staff considers that these inspections, though limited in scope, provide an important added level of confidence in the continued integrity of the containment boundary. The Prairie Island containment vessels are free-standing steel structures designed for the peak pressure of the design basis accident and low leakage. A concrete shield building surrounds the containment vessel, providing a shield building annulus between the two structures. Penetrations of the containment vessel for piping, electrical conductors, ducts and access hatches are provided with double barriers against leakage. The NRC staff also notes that due to the free-standing design of the containment structure, the vessel shell and penetrations are accessible for inspection from both inside containment and outside in the shield building annulus.

The NRC staff has also made use of the information in a draft staff report, NUREG-1493, "Performance-Based Containment Leak-Test Program," which provides the technical justification for the present Appendix J rulemaking effort which also includes a 10-year test interval for Type A tests. The integrated leakage rate test, or Type A test, measures overall containment leakage. However, operating experience

with all types of containments used in this country demonstrates that essentially all containment leakage can be detected by local leakage rate tests (Type B and C). According to results given in NUREG-1493, out of 180 ILRT reports covering 110 individual reactors and approximately 770 years of operating history, only 5 ILRT failures were found which local leakage rate testing could not detect. This is 3% of all failures. This study agrees well with previous NRC staff studies which show that Type B and C testing can detect a very large percentage of containment leaks. The Prairie Island Nuclear Generating Plant, Unit No. 2, experience has also been consistent with these results.

The Nuclear Management and Resources Council (NUMARC), now the Nuclear Energy Institute (NEI), collected and provided the NRC staff with summaries of data to assist in the Appendix J rulemaking effort. NUMARC collected results of 144 ILRTs from 33 units; 23 ILRTs exceeded $1L_a$. Of these, only nine were not type B or C leakage penalties. The NEI data also added another perspective. The NEI data show that in about one-third of the cases exceeding allowable leakage, the as-found leakage was less than $2L_a$; in one case the leakage was found to be approximately $2L_a$; in one case the as-found leakage was less than $3L_a$; one case approached $10L_a$; and in one case the leakage was found to be approximately $21L_a$. For about half of the failed ILRTs the as-found leakage was not quantified. These data show that, for those ILRTs for which the leakage was quantified, the leakage values are small in comparison to the leakage value at which the risk to the public starts to increase over the value of risk corresponding to L_a (approximately $200L_a$, as discussed in NUREG-1493). Therefore, based on these considerations, it is unlikely that an extension of one cycle for the performance of the Appendix J, Type A test at Prairie Island Nuclear Generating Plant, Unit No. 2, would result in significant degradation of the overall containment integrity. As a result, the application of the regulation in these particular circumstances is not necessary to achieve the underlying purpose of the rule. Based on the generic and plant-specific data, the NRC staff finds the basis for the licensee's proposed one-time schedular exemption to allow an extension of one cycle for the performance of the Appendix J, Type A test, provided that the general containment inspection is performed, to be acceptable.

Pursuant to 10 CFR 51.32, the Commission has determined that granting this exemption will not have a significant effect on the quality of the human environment (60 FR 18428).

This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 12th day of April 1995.

For the Nuclear Regulatory Commission.

Elinor G. Adensam,

Acting Director, Division of Reactor Projects III/IV, Office of Nuclear Reactor Regulation.

[FR Doc. 95-9637 Filed 4-18-95; 8:45 am]

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Advisory Committee on Medical Uses of Isotopes: Meeting Notice

AGENCY: U.S. Nuclear Regulatory Commission.

ACTION: Notice of Meeting.

SUMMARY: The U.S. Nuclear Regulatory Commission will convene its next regular meeting of the Advisory Committee on the Medical Uses of Isotopes (ACMUI) on May 11 and 12, 1995. Topics of discussion will include: (1) Brachytherapy issues; (2) Guidance documents for the final Radiopharmacy Rule; (3) Prostate implant procedures; (4) National Program Review II; (5) Training and experience of authorized users to allow exemptions to Subpart J; (6) Dose ranges in written directives; (7) Petition to review the final Radiopharmacy Rule; (8) Information from NIST on Sr-90 calibration errors for eye applicators; (9) Revisions to Regulatory Guide 10.8; (10) Status of implementation of the Quality Management rule; (11) Update on the study of the medical use program by the National Academy of Science; (12) Summary of "Business Process Reengineering;" (13) Update on rulemakings: "Medical Administration of Radiation and Radioactive Materials," "Release of Patients Containing Radiopharmaceuticals or Permanent Implants," and "Administration of Byproduct Material or Radiation from Byproduct Material to Patients who may be Pregnant or Nursing."

DATES: The meeting will begin at 8 a.m. on May 11 and 12, 1995.

ADDRESSES: U.S. Nuclear Regulatory Commission, Two White Flint North, 11545 Rockville Pike, Room T2B3, Rockville, MD 20852-2738.

FOR FURTHER INFORMATION CONTACT: Josephine M. Piccone, Ph.D., U.S. Nuclear Regulatory Commission, Office of Nuclear Material Safety and Safeguards, MS T8F5, Washington, DC 20555, Telephone (301) 415-7270. For

administrative information, contact Torre Taylor, (301) 415-7900.

Conduct of the Meeting

Barry Siegel, M.D., will chair the meeting. Dr. Siegel will conduct the meeting in a manner that will facilitate the orderly conduct of business. The following procedures apply to public participation in the meeting:

1. Persons who wish to provide a written statement should submit a reproducible copy to Josephine M. Piccone (address listed above). Comments must be received by May 3, 1995, to ensure consideration at the meeting. The transcript of the meeting will be kept open until May 19, 1995, for inclusion of written comments.

2. Persons who wish to make oral statements should inform Dr. Piccone in writing, by May 3, 1995. Statements must pertain to the topics on the agenda for the meeting. The Chairman will rule on requests to make oral statements. Members of the public will be permitted to make oral statements if time permits. Permission to make oral statements will be based on the order in which requests are received. In general, oral statements will be limited to approximately 5 minutes. Oral statements must be supplemented by detailed written statements for the record. Rulings on who may speak, the order of presentation, and time allotments may be obtained by calling Dr. Piccone, (301) 415-7270, between 8 a.m. and 4 p.m., EST, on May 9, 1995.

3. At the meeting, questions from attendees other than committee members, NRC consultants, and NRC staff will be permitted at the discretion of the Chairman.

4. The transcript, minutes of the meeting, and written comments will be available for inspection, and copying, for a fee, at the NRC Public Document Room, 2120 L Street NW, Lower Level, Washington, DC 20555 (202) 634-3273, on or about May 26, 1995.

5. Seating for the public will be on a first-come, first-served basis.

This meeting will be held in accordance with the Atomic Energy Act of 1954, as amended (primarily Section 161a); the Federal Advisory Committee Act (5 U.S.C. App); and the Commission's regulations in Title 10, *Code of Federal Regulations*, Part 7.

Dated: April 13, 1995.

Andrew L. Bates,

Advisory Committee Management Officer.

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