

Signed at Washington, D.C., this 7th day of April 1995.

Victor J. Trunzo,

Program Manager, Policy and Reemployment Services, Office of Trade Adjustment Assistance.

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NATIONAL AERONAUTICS AND SPACE ADMINISTRATION

[Notice 95-028]

NASA Advisory Council, Earth Systems Science and Applications Advisory Committee (ESSAAC); Meeting

AGENCY: National Aeronautics and Space Administration.

ACTION: Notice of Meeting.

SUMMARY: In accordance with the Federal Advisory Committee Act, Public Law 92-463, as amended, the National Aeronautics and Space Administration announces a forthcoming meeting of the NASA Advisory Council, Earth Systems Science and Applications Advisory Committee.

DATES: May 17, 1995, 8:30 a.m. to 5:30 p.m.; and May 18, 1995, 8:30 a.m. to 5:30 p.m..

ADDRESSES: National Aeronautics and Space Administration, MIC-7 Conference Room, 300 E Street, S.W., Washington, DC 20546.

FOR FURTHER INFORMATION CONTACT:

Dr. Robert A. Schiffer, Code YS, National Aeronautics and Space Administration, Washington, DC 20546, (202) 358-1876.

SUPPLEMENTARY INFORMATION: The meeting will be open to the public up to the seating capacity of the room. The provisional agenda for the meeting is as follows: NASA Response to ESSAAC Recommendations; impact of NASA Streamlining on MTPE Science Program; the MTPE Strategic Plan; committee discussion; and findings, conclusions, and recommendations.

It is imperative that the meeting be held on these dates to accommodate the scheduling priorities of the key participants. Visitors will be requested to sign a visitor's register.

Dated: April 12, 1995.

Danalee Green,

Chief, Management Controls Office.

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-412]

Duquesne Light Company; Ohio Edison Company; The Cleveland Electric Illuminating Company; The Toledo Edison Company; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-73, issued to Duquesne Light Company, et al., (the licensee), for operation of the Beaver Valley Power Station, Unit 2 (BVPS-2), located in Beaver County, Pennsylvania.

The proposed amendment would revise Technical Specification (TS) 4.6.2.2.d to delete the reference to the specific test acceptance criteria for the Containment Recirculation Spray Pumps and replace the specific test acceptance criteria with reference to the requirements of the Inservice Testing (IST) Program. In addition, the 18-month test frequency would be replaced with the test frequency requirements specified in the IST Program. The current footnote (1) pertaining to the performance of recirculation spray pump 2RSS*P21A would be deleted.

This proposed amendment is requested to be processed as an exigent TS change in accordance with 10 CFR 50.91(a)(6). Exigent processing is being requested because BVPS-2 entered Mode 5 for the purpose of performing its fifth refueling outage on March 25, 1995, and upon completion of testing of Recirculation Spray Pump 2RSS*P21A, the licensee concluded that this pump failed to satisfy the specific test acceptance criteria in TS 4.6.2.2.d. Pump disassembly for inspection and repairs commenced on April 5, 1995. The pump is scheduled to be reassembled and flow tested by April 12, 1995. BVPS-2 is currently scheduled to enter Mode 4 on May 4, 1995, at which time pump 2RSS*P21A is required to be operable. If the pump does not meet the specific test acceptance criteria currently in TS 4.6.2.2.d at that time, BVPS-2 will be prohibited from entry into Mode 4. With the proposed revision to TS 4.6.2.2.d, the actual performance of pump 2RSS*P21A could then be evaluated against accident analysis assumptions and the pump's acceptance criteria could then be revised under the provisions of 10 CFR 50.59 to establish

IST Program requirements that would continue to maintain the plant within the accident analysis assumptions. The licensee could not have foreseen this event since the pump's performance could not be tested until the plant entered Mode 5 on March 25, 1995.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

Pursuant to 10 CFR 50.91(a)(6) for amendments to be granted under exigent circumstances, the NRC staff must determine that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the change involve a significant increase in the probability or consequences of an accident previously evaluated?

The change does not result in a modification to plant equipment nor does it affect the manner in which the plant is operated. The Recirculation Spray System (RSS) pumps are normally in a standby condition and only operate during accident mitigation. Since the physical plant equipment and operating practices are not changed, as noted above, there is no change in the probability of an accident previously evaluated.

The proposed change will not lower the pump performance operability criteria for the RSS pumps. The required values for developed pump head and flow will continue to satisfy accident mitigation requirements and will be maintained and controlled in the Beaver Valley Power Station (BVPS) Unit No. 2 Inservice Testing (IST) Program.

Since the proposed change does not lower the RSS pump performance acceptance criteria, the containment depressurization system will continue to meet its design basis requirements. The proposed change will not impose additional challenges to the containment structure in terms of peak pressure. The calculated offsite dose consequences of a design basis accident (DBA) will remain unchanged since the one hour release duration remains unchanged. The ability of the RSS pumps to provide sufficient long term core cooling also remains unchanged. The proposed change in the RSS pump surveillance interval from 18 months