

Dated: April 11, 1995.

Sam Duraiswamy,

Chief, Nuclear Reactors Branch.

[FR Doc. 95-9370 Filed 4-14-95; 8:45 am]

BILLING CODE 7590-01-M

Advisory Committee on Reactor Safeguards; Subcommittee Meeting on Reactor Fuels; Notice of Meeting

The ACRS Subcommittee on Reactor Fuels will hold a meeting on May 3, 1995, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Wednesday, May 3, 1995—8:30 a.m. until the conclusion of business.

The Subcommittee will begin review of the NRC and industry programs being conducted to address issues associated with the use of high-burnup fuel. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittee, along with any of its consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Subcommittee will then hear presentations by and hold discussions with representatives of the NRC staff, its consultants, and other interested persons regarding this review.

Further information regarding topics to be discussed, whether the meeting has been cancelled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor can be obtained by contacting the cognizant ACRS staff engineer, Dr. Medhat El-Zeftawy (telephone 301/415-6889) between 7:30 a.m. and 4:15 p.m. (EDT). Persons planning to attend this meeting

are urged to contact the above named individual one or two working days prior to the meeting to be advised of any potential changes in the proposed agenda, etc., that may have occurred.

Dated: April 11, 1995.

Sam Duraiswamy,

Chief, Nuclear Reactors Branch.

[FR Doc. 95-9371 Filed 4-14-95; 8:45 am]

BILLING CODE 7590-01-M

[Docket No. 50-219]

Issuance of Amendment to Facility Operating License; GPU Nuclear Corporation

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment No. 179 to Facility Operating License No. DPR-16 issued to GPU Nuclear Corporation (the licensee), which revised the Technical Specifications for operation of the Oyster Creek Nuclear Generating Station located in Ocean County, New Jersey. The amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

The amendment revises Technical Specification 5.3.1.E to allow 2645 fuel assemblies to be stored in the fuel pool. This is an increase of 45 fuel assemblies from the current limit of 2600 fuel assemblies. The 45 additional storage locations currently exist in the racks in the fuel pool.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity for Hearing in connection with this action was published in the **Federal Register** on December 20, 1994 (59 FR 65542). No request for a hearing or petition for leave to intervene was filed following this notice.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of the amendment will not have a significant effect on the quality of the human environment (60 FR 17373).

For further details with respect to the action see (1) the application for

amendment dated November 25, 1994, as supplemented February 15, 1995, and (2) Amendment No. 179 to License No. DPR-16, (3) the Commission's related Safety Evaluation, and (4) the Commission's Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street NW., Washington, DC, and at the local public document room located at the Ocean County Library, Reference Department, 101 Washington Street, Toms River, NJ 08753.

Dated at Rockville, Maryland, this 10th day of April 1995.

For the Nuclear Regulatory Commission.

Alexander W. Dromerick, Sr.,

Project Manager, Project Directorate I-3, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

[FR Doc. 95-9372 Filed 4-14-95; 8:45 am]

BILLING CODE 7590-01-M

[Docket No. 50-322; License No. NPF-82]

In the Matter of: Long Island Power Authority (Shoreham Nuclear Power Station, Unit 1); Order Terminating the Facility Operating License

I

Long Island Power Authority (LIPA or the licensee) is the holder of Facility Operating License No. NPF-82, issued by the U.S. Nuclear Regulatory Commission (NRC) pursuant to part 50 of Title 10 of the Code of Federal Regulations (10 CFR) on April 21, 1989. The license was amended on June 14, 1991, to remove the authority to operate the Shoreham Nuclear Power Station (SNPS), in accordance with conditions specified therein, and authorized the possession only for SNPS. The facility is located on LIPA's site in the town of Brookhaven, Suffolk County, New York.

II

On February 28, 1989, Long Island Lighting Company (LILCO), the former licensee, entered into an agreement with the State of New York to transfer its Shoreham assets to an entity of the State (LIPA) for decommissioning. LILCO, however, continued to pursue with NRC its request for a full-power license to operate its Shoreham plant. On April 21, 1989, NRC issued to LILCO a Facility Operating License No. NPF-82, which allowed full-power operation of the Shoreham plant. On June 28, 1989, LILCO's shareholders ratified LILCO's agreement with the State. Consistent with the terms of the settlement agreement, which prohibits further operation of the Shoreham facility,

LILCO defueled the reactor and reduced its staff. On February 29, 1992, NRC approved the transfer of the Shoreham license to LIPA.

In a letter dated June 27, 1991 (SNRC-1818), the former licensee, LILCO requested, in accordance with the requirement of 10 CFR 50.82(a), approval for termination of SNPS License No. NPF-82. The application for license termination was preceded by a proposed decommissioning plan (DP) that was submitted to NRC on December 29, 1990.

In June 1992, the NRC approved the licensee's DP, which described the planned dismantlement and decontamination methods that would be used to reduce contamination at the facility to meet the unrestricted release criteria. By an agreement between the Philadelphia Electric Company (PECo) and LIPA, the slightly irradiated fuel stored in the SNPS spent fuel pool was transferred to the Limerick Generating Station for reuse. The fuel transfers began in September 1993, and were completed in June 1994.

The dismantlement and decontamination tasks necessary for decommissioning SNPS, began in June 1992, and were completed in August 1994. All contaminated waste generated during the decommissioning of SNPS has been removed from the site.

III

The SNPS proposed DP was approved by an Order dated June 11, 1992. Approximately 602 curies of radioactive material were present in the RPV and internal components; 1370 curies in the control rod blades, local-power-range monitors, and antimony/beryllium pins; 457 millicuries in the reactor bioshield wall; and 3 millicuries in contaminated systems and areas. Approximately 593 millicuries were present in the spent fuel storage pool water (SFSP) and on the surfaces of the SFSP and fuel storage racks.

The SNPS was decommissioned in accordance with the approved DP, as supplemented. The licensee decommissioned SNPS by dismantlement and decontamination. The activated and contaminated reactor vessel internals were removed and disposed of as radioactive waste. Activated portions of the reactor pressure vessel (RPV) that exceeded unrestricted release criteria, were segmented, packaged, and disposed as radioactive waste. Large contaminated components and portions of plant systems and structures that could not be decontaminated effectively were segmented, as necessary, and packaged and shipped offsite for volume

reduction and/or direct disposal at licensed low-level waste disposal facilities. Contaminated water was processed and released in accordance with approved release requirements, and activated portions of the reactor bioshield wall exceeding unrestricted release criteria were removed and disposed as low-level radioactive waste.

The DP, as supplemented, contained the licensee's Termination Survey Plan (Plan). The Plan described the methods used to demonstrate compliance with the existing NRC unrestricted release criteria. The guidelines used by the licensee for residual radioactivity at the SNPS are consistent with the values provided in Table 1, of Regulatory Guide 1.86, which establishes acceptable residual surface contamination levels. NRC approved alternative contamination limits for iron-55 and tritium above those specified in Regulatory Guide 1.86. These alternative criteria were presented to the Commission in SECY 94-145 and increased the allowable residual average and maximum total residual beta activity levels for iron-55 and tritium from 5,000 average total and 15,000 maximum total (fixed plus removable) disintegrations per minute (dpm)/100 square centimeters to 200,000 average total and 600,000 maximum total dpm/100 square centimeters, respectively. This permitted the licensee to safely retain on site major portions of the reactor bioshield wall that did not exceed the gamma dose rate criterion or the surface contamination limits for other isotopes, but which would have required offsite disposal under the original iron-55 and tritium surface contamination limits of Regulatory Guide 1.86. A concentration limit for cobalt-60 in soil and other bulk materials of 8 picocuries per gram was also established. An average gamma dose rate criterion of 5 uR per hour above background, at a distance of 1 meter from accessible surfaces, and an individual gamma exposure rates are not to exceed 10 uR per hour above background at 1 meter were used. In addition, the licensee's Plan also described the technical methods that would be used for the planning and design of the Termination Survey. The methods used by the licensee for the planning and design of the Termination Survey were derived from regulatory guidance contained in the NRC Regulatory Guide 1.86 and draft NUREG/CR-5849.

The licensee's Termination Plan was implemented at the individual survey unit level. The licensee established the following three survey units: (1) Structures, which included building

interiors; (2) plant systems; and (3) outdoor areas. The licensee used a phased approach to conduct the actual termination surveys. Phase 1 of the termination surveys included surveys of the main turbine internals, turbine building, site and grounds, and the exterior of buildings. Phases 2 and 3 of the termination surveys included portions of the reactor and radwaste buildings and the suppression pool. Phase 4 (final phase) of the termination survey included those portions of the reactor and spent fuel pool used to support irradiated fuel storage, before the fuel's removal from the site. The licensee completed the Termination Surveys in August 1994.

The licensee submitted phases 1, 2, and 3 of the Final Termination Survey Reports to NRC in September 1993, February 1994, and June 1994, respectively. Phase 4 of the Final Termination Report was submitted to NRC for review in October 1994. The Oak Ridge Institute for Science and Education (ORISE) performed independent confirmatory surveys at SNPS for NRC.

Based on the licensee's completion of the decommissioning tasks at the SNPS, as described in the approved DP, and supplements, the staff's review of the licensee's Final Termination Survey Reports, and the results of the ORISE confirmatory surveys, the staff has determined that the decommissioning of the SNPS is complete, that the residual radioactivity levels have been reduced in accordance with NRC release criteria, and that the facility and the site are suitable to be released for unrestricted use.

IV

Accordingly, pursuant to sections 103, 161b, 161i, and 161o of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR 2.202 and 10 CFR 50.82(f), NRC terminates License No. NPF-82.

Any person adversely affected by this Order may request a hearing within 20 days of its issuance. Any request for a hearing shall be submitted to the Secretary, U.S. Nuclear Regulatory Commission, ATTN: Chief, Docketing and Service Section, Washington, DC 20555. Copies of the hearing request shall also be sent to the Director, Office Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555, to the Assistant General Counsel for Hearings and Enforcement at the same address, to the Regional Administrator, NRC Region I, 475 Allendale Road, King of Prussia, PA 19406, and to the licensee. If such a person requests a hearing, that person

shall set forth with particularity the manner in which his interest is adversely affected by this Order and shall address the criteria set forth in 10 CFR 2.714(d). The Commission will issue an Order designating the time and place of the hearing.

If a hearing is granted, the issue to be heard is whether the licensee complied with the Commission-approved decommissioning plan.

In the absence of any request for hearing, the provisions specified in Section IV above shall be final 20 days from the date of this Order without further order or proceedings.

For further details with respect to this action, see: (1) The licensee's request to terminate the SNPS license presented in letters dated June 27, 1991 (SNRC-1818), and August 4, 1994 (LSNRC-2178); (2) the Commission's Order approving decommissioning dated June 11, 1992; (3) the licensee's Termination Survey Final Report, Phase 1 (LSNRC-2101), dated September 30, 1993; the licensee's Termination Survey Final Report, Phase 2 (LSNRC-2144), dated February 4, 1994; the licensee's Termination Survey Final Report, Phase 3 (LSNRC-2173), dated June 14, 1994; the licensee's Termination Survey Final Report, Phase 4 (LSNRC-2184), dated October 12, 1994; and (4) the ORISE Final Confirmatory Reports dated July 1993, September 1994, and February 1995. These documents are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2102 L Street, NW., Washington, DC 20555, and at the Shoreham Wading River Public Library, Route 25A, Shoreham, NY 11786. Copies may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Waste Management.

Dated at Rockville, Maryland, this 11th day of April, 1995.

For the Nuclear Regulatory Commission.

John T. Greeves,

*Director, Division of Waste Management,
Office of Nuclear Material Safety and
Safeguards.*

[FR Doc. 95-9373 Filed 4-14-95; 8:45 am]

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SECURITIES AND EXCHANGE COMMISSION

Request Under Review by Office of Management and Budget

Acting Agency Clearance Officer: David
T. Copenhafer, (202) 942-8800.
Upon written request copies available
from: Securities and Exchange

Commission, Office of Filings and
Information Services, 450 Fifth
Street NW., Washington, DC 20549.
Extension: Rule 15a-6, File No. 270-
329.

Notice is hereby given that pursuant to the Paperwork Reduction Act of 1980 (44 U.S.C. 3501 *et seq.*), the Securities and Exchange Commission has submitted a request for extension of OMB approval to Rule 15a-6 [17 CFR 240.15a-6] under the Securities Exchange Act of 1934 (15 U.S.C. 78a *et seq.*).

Rule 15a-6 provides, among other things, an exemption from broker-dealer registration for foreign broker-dealers that effect trades with or for U.S. institutional investors through a U.S. registered broker-dealer, provided that the U.S. broker-dealer obtains certain information about, and consents to service of process from, the personnel of the foreign broker-dealer involved in such transactions, and maintains certain records in connection therewith. It is estimated that approximately 2,000 respondents will incur an average burden of three hours per year to comply with this rule.

Direct general comments to the Clearance Officer for the Securities and Exchange Commission at the address below. Direct any comments concerning the accuracy of the estimated average burden hours for compliance with the Commission rules and forms to David T. Copenhafer, Acting Director, Office of Information Technology, Securities and Exchange Commission, 450 Fifth Street NW., Washington, D.C. 20549 and the Clearance Officer for the Securities and Exchange Commission, Office of Management and Budget, Paperwork Reduction Project number 3235-0371, Room 3208, New Executive Office Building, Washington, D.C. 20503.

Dated: April 6, 1995.

Margaret H. McFarland,

Deputy Secretary.

[FR Doc. 95-9395 Filed 4-14-95; 8:45 am]

BILLING CODE 8010-01-M

[Release No. 34-35581; File No. SR-BSE-94-05]

Self-Regulatory Organizations; the Boston Stock Exchange Inc.; Order Approving Proposed Rule Change Relating to Implementation of a Three- Day Settlement Standard

April 7, 1995.

On February 21, 1995, the Boston Stock Exchange Incorporated ("BSE") filed a proposed rule change (File No. SR-BSE-95-05) with the Securities and Exchange Commission ("Commission")

pursuant to Section 19(b) of the Securities Exchange Act of 1934 ("Act").¹ Notice of the proposal was published in the **Federal Register** on March 8, 1995 to solicit comments from interested persons.² As discussed below, this order approves the proposed rule change.

I. Description

In October 1993, the Commission adopted Rule 15c6-1 under the Act³ which establishes three business days after the trade date ("T+3"), instead of five business days ("T+5"), as the standard settlement cycle for most securities transactions. The rule will become effective on June 7, 1995.⁴ Several of the BSE's rules are interrelated with the standard settlement time frame. The purpose of the proposed rule change is to amend BSE's rules to be consistent with a T+3 settlement standard for securities transactions.

Chapter II, Section 6 of BSE's definition of "Regular Way," will be amended to require settlement of regular way transactions on the third business day after the trade.⁵ Buyer's or seller's option trades will settle between four business days and 180 days following the contract date except that BSE may provide otherwise in specific issues or classes of securities. Next day trades will settle on the first or second business day following the date of the contract. Under Chapter X, Section 1, securities will trade without (*i.e.*, "ex") any dividend, right, or privilege on the second full business day preceding the record date except that when the record date is on a holiday the securities will trade "ex" on the third preceding full business day.

The proposed rule change also will amend Chapter XV, Section 14, "Claims and Reports against Specialists," to shorten the time periods in which members can file claims of erroneous or omitted transactions against specialists. Claims regarding lack of a comparison of a reported transaction must be made within three days of the original trade date. Claims regarding the omission of reports and erroneous trade comparisons will have to be within five business days. The proposed rule

¹ 15 U.S.C. 78s(b) (1988).

² Securities Exchange Act Release No. 35422 (February 28, 1995), 60 FR 12793.

³ 17 CFR 240.15c6-1 (1994).

⁴ Securities Exchange Act Release Nos. 33023 (October 6, 1993), 58 FR 52891 (adopting Rule 15c6-1) and 34952 (November 9, 1994), 59 FR 59137 (changing effective date from June 1, 1995, to June 7, 1995).

⁵ References to five-day delivery contained in Section 5 will be deleted.