

[NAFTA-00319]

**Woodward Governor Co., Stevens Point, WI.; Amended Certification Regarding Eligibility to Apply for NAFTA Transitional Adjustment Assistance**

In accordance with Section 250(a), Subchapter D, Chapter 2, Title II, of the Trade Act of 1974, as amended (19 U.S.C. 2273), the Department of Labor issued a Certification for NAFTA Transitional Adjustment Assistance on February 3, 1995, applicable to all workers of the Aircraft Controls Group of the subject firm in Stevens Point, Wisconsin. The notice was published in the **Federal Register** on February 14, 1995 (60 FR 8416).

At the request of the State Agency and the company, the Department reviewed the certification for workers of the subject firm. New findings show that some production was in hydromatic controls. The workers were not entirely separately identifiable by product line and the plant will close in 1995. Accordingly, the Department is amending the certification to include all workers at Woodward Governor Company in Stevens Point, Wisconsin.

The intent of the Department's certification is to include all workers who were adversely affected at Woodward Governor Company in Stevens Point, Wisconsin by increased imports.

The amended notice applicable to NAFTA-00319 is hereby issued as follows:

All workers of the Woodward Governor Company, Stevens Point, Wisconsin who became totally or partially separated from employment on or after December 27, 1993 are eligible to apply for NAFTA-TAA under Section 250 of the Trade Act of 1974.

Signed at Washington, D.C., this 3rd day of March 1995.

**Victor J. Trunzo,**

*Program Manager, Policy and Reemployment Services, Office of Trade Adjustment Assistance.*

[FR Doc. 95-6655 Filed 3-16-95; 8:45 am]

BILLING CODE 4510-30-M

**NATIONAL SCIENCE FOUNDATION****Advisory Committee for Biological Sciences (BIO); Notice of Meeting**

In accordance with the Federal Advisory Committee Act (Pub. L. 92-463, as amended), the National Science Foundation announces the following meeting:

*Name:* Advisory Committee for Biological Sciences (BIO) (1110).

*Date and Time:* April 3, 1995; 8:45 a.m.–6 p.m.; April 4, 1995; 8:45 a.m.–12 Noon.

*Place:* National Science Foundation, 4201 Wilson Blvd., Arlington, VA 22230, Room 1235.

*Type of Meeting:* Open.

*Contact Person:* Dr. Mary E. Clutter, Assistance Director, Biological Sciences, Room 605, National Science Foundation, 4201 Wilson Blvd., Arlington, VA 22230, Tel No.: (703) 306-1400.

*Minutes:* May be obtained from the contact person listed above.

*Purpose of Meeting:* The Advisory Committee for BIO provides advice, recommendations, and oversight concerning major program emphases, directions, and goals for the research-related activities of the divisions that make up BIO.

*Agenda:* NSF and BIO strategic planning; integration of research and education.

Dated: March 14, 1995.

**M. Rebecca Winkler,**

*Committee Management Officer.*

[FR Doc. 95-6679 Filed 3-16-95; 8:45 am]

BILLING CODE 7555-01-M

**NUCLEAR REGULATORY COMMISSION**

[Docket No. 30-16055-ML-Ren, ASLBP No. 95-707-02-ML-Ren, (Source Material License No. 34-19089-01)]

**Atomic Safety and Licensing Board Panel; Notice of Hearing**

March 13, 1995.

In the Matter of Advanced Medical Systems, Inc. Cleveland, Ohio.

Notice is hereby given that, by Memorandum and Order dated March 10, 1995, the Presiding Officer in this proceeding has granted the hearing requests of the Northeast Ohio Regional Sewer District (dated December 29, 1994) and the City of Cleveland, Ohio (dated January 13, 1995), and has conditionally granted the participation of the Cuyahoga County Local Emergency Planning Committee in a hearing regarding the license renewal application of Advanced Medical Systems, Inc. for its facility located at 1020 London Road in Cleveland, Ohio. Advanced Medical Systems, Inc. seeks continued permission from the Nuclear Regulatory Commission to possess various quantities of radioactive materials for use in its manufacture of medically related devices. The hearing will involve the sufficiency of the renewal application.

This proceeding will be conducted under the Commission's Informal Hearing Procedures for Adjudications in Materials and Operator Licensing Proceedings, set forth in 10 CFR part 2, subpart L. Further details appear in Statement of Considerations, Informal

Hearing Procedures for Materials Licensing Adjudications, 54 Fed. Reg. 8269 (February 28, 1989) and the March 10, 1995 Memorandum and Order referenced above. Documents relating to this proceeding are available for public inspection and copying at the Commission's Public Document Room, Gelman Building, 2120 L St. NW., Washington, DC.

Advanced Medical Systems, Inc., the Northeast Ohio Regional Sewer District, the City of Cleveland, and the NRC Staff are parties to this proceeding. The Cuyahoga County Local Emergency Planning Committee may participate in this proceeding under the provisions of 10 CFR 2.1211(b) upon submission to the Presiding Officer (and service on the parties) of an affidavit of a Cuyahoga County official attesting that the Local Emergency Planning Committee is representing the County's interest in this matter. If admitted as a representative of an interested county, the Local Emergency Planning Committee's participation shall be limited to the extent allowed by 10 CFR 2.1211(b).

In accordance with 10 CFR 2.1205(l)(4), any person whose interest may be affected by this proceeding may, within 30 days of publication of this Notice, file a petition for leave to intervene. Such petition must identify (1) the interest of the petitioner in the proceeding, (2) how that interest may be affected by the results of the proceeding, with particular reference to the factors set out in 10 CFR 2.1205(g), (3) the petitioner's areas of concern about the licensing activity which must be germane to the subject matter of the proceeding, and (4) the circumstances establishing that the petition is timely and that the petitioner has the requisite standing to intervene in the hearing.

Each petition must be submitted to the Secretary of the Commission, ATTN: Chief, Docketing and Services Branch, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Copies should be served upon the Presiding Officer; the Special Assistant; the Assistant General Counsel for Hearings and Enforcement; the Senior Attorney, Atomic Safety and Licensing Board Panel; and the Executive Director for Operations, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Copies should also be served on the Licensee, through its attorney, Henry E. Billingsley, II, Arter and Hadden, 1100 Huntington Building, 925 Euclid Avenue, Cleveland, Ohio 44115; and the other parties through Thomas E. Lenhart, Assistant General Counsel, Northeast Ohio Regional Sewer District, 3826 Euclid Avenue, Cleveland, Ohio

44115; and Martha R. McCorkle, Assistant Director of Law, City of Cleveland Department of Law, Room 106, City Hall, 601 Lakeside Avenue, Cleveland, Ohio 44114. Pursuant to 10 CFR 2.1205(j)(2), any party may file an answer to a petition to intervene within 10 days of service of such petition (15 days in the case of the NRC Staff).

Pursuant to 10 CFR 2.1211(a), any member of the public who is not a party to this proceeding may make a written statement in order to express his or her views of the issues involved in this license renewal proceeding. These statements are not evidence and do not become part of the decisional record under 10 CFR 2.1251(c). Written statements should be submitted to the Secretary of the Commission, ATTN: Chief, Docketing and Services Branch, U.S. Nuclear Regulatory Commission, Washington, DC 20555.

Dated: March 13, 1995.

**Marshall E. Miller,**

*Presiding Officer, Administrative Judge.*

[FR Doc. 95-6617 Filed 3-16-95; 8:45 am]

BILLING CODE 7590-01-M

#### [Docket Nos. 50-373 50-374

#### Exemption

In the Matter of Commonwealth Edison Co., LaSalle County Station, Units 1 and 2.

#### I

Commonwealth Edison Company (ComEd, the licensee) is the holder of Facility Operating License Nos. NPF-11 and NPF-18, which authorize operation of the LaSalle County Station, Units 1 and 2 (the facility), at a steady state power level not in excess of 3323 megawatts thermal. The facility consists of two boiling water reactors at the licensee's site located in LaSalle County, Illinois. The licenses provide, among other things, that they are subject to all rules, regulations, and orders of the U.S. Nuclear Regulatory Commission (the Commission) now or hereafter in effect.

#### II

Section III.A.6(b) of Appendix J to 10 CFR Part 50 states the following in regard to performing Overall Integrated Containment Leakage Rate (Type A) Tests (ILRT):

If two consecutive periodic Type A tests fail to meet the applicable acceptance criteria in III.A.5(b), notwithstanding the periodic retest schedule of III.D., a Type A test shall be performed at each plant shutdown for refueling or approximately every 18 months, whichever occurs first, until two consecutive Type A tests meet the acceptance criteria in

III.A.5(b), after which time the retest schedule specified in III.D. may be resumed.

The Type A tests performed during the first, third and fourth refueling outages for LaSalle County Station, Unit 2, were considered to be failures in the "as-found" condition due to penalties incurred as a result of leakage measured in Type B and C local leak rate tests (LLRT). Pursuant to Section III.A.6(b) of Appendix J, Type A testing was performed during the fifth refueling outage for LaSalle County Station, Unit 2, in December 1993. That Type A test satisfied the "as-found" acceptance criteria. Section III.A.6(b) of Appendix J requires an additional Type A test during the sixth refueling outage, currently scheduled for February 1995, in order to fulfill the condition of two consecutive successful tests prior to resuming the Type A test interval of Section III.D.

As an alternative to performing the required Type A test, the licensee has submitted a Corrective Action Plan to address excessive local leakage in accordance with the guidance provided in NRC Information Notice 85-71, "Containment Integrated Leak Rate Tests," dated August 22, 1985. The Corrective Action Plan is in lieu of the increased test frequency required by Section III.A.6(b) and, therefore, an exemption from this requirement is needed.

Section III.D.1(a) of Appendix J requires " \* \* \* a set of three Type A tests shall be performed, at approximately equal intervals during each 10-year service period. The third test of each set shall be conducted when the plant is shutdown for the 10-year plant inservice inspections." The last refueling outage for Unit 2 during the first 10-year inservice inspection period is the sixth refueling outage scheduled for February 1995. Therefore, in addition to the requirements for additional testing specified in Section III.A.6(b), a Type A test is required during the upcoming Unit 2 refueling outage as a result of the periodic retest schedule contained in Section III.D.1(a). To address the short-term desire not to perform a Type A test during the sixth refueling outage for Unit 2 and avoid potential future problems, the licensee has requested an exemption from this requirement such that future Type A test would not need to coincide with the end of 10-year inservice inspection periods.

The NRC may grant exemptions from the requirements of the regulations, pursuant to 10 CFR 50.12, that (1) are authorized by law, will not present an undue risk to the public health and

safety, and are consistent with the common defense and security; and (2) present special circumstances. Section 50.12(a)(2) of 10 CFR Part 50 describes special circumstances as including cases that would not serve the underlying purpose of the rule or are not necessary to achieve the underlying purpose of the rule.

#### III

The underlying purpose of the requirements in Appendix J is to ensure that containment leakage remains below criteria established to limit the release of radioactive materials in the event of a design basis accident. The Type A test is defined in 10 CFR Part 50, Appendix J, Section II.F, as a "test intended to measure the primary reactor containment overall integrated leakage rate (1) after the containment has been completed and is ready for operation, and (2) at periodic intervals thereafter." Containment leakage is measured during the periodic testing required by Section III.D.1(a) and the additional testing requirements of Section III.A.6 if the measured leakage exceeds the established limits. The testing and other requirements contained in Appendix J ensure that leakage from the containment structure and penetrations remain below the acceptance criteria.

The licensee conducted four ILRTs during the first 10-year service period for Unit 1. For Unit 2, ILRTs were performed during the first, third, fourth, and fifth refueling outages. The Type A test history for Unit 2 is that the measured leakage rates for Type B and C penetrations, when added to the measured results from the Type A test, resulted in an "as-found" integrated leakage rate above the acceptance criteria. These test failures were the direct result of leakage penalties from Type B and C LLRTs.

Leakage from specific containment penetrations that have been major contributors to the failure of the integrated leakage rate acceptance criteria for Unit 2 have been identified. These leakage paths include isolation valves associated with the drywell equipment and floor drain sumps, reactor water cleanup suction, transversing incore probe air purge supply, residual heat removal shutdown cooling return, hydrogen recombiners, and primary containment chilled water supply. The leakage associated with the reactor water cleanup suction penetration provided the overwhelming contribution of local leakage penalty that resulted in the unsuccessful test during the fourth refueling outage. Leakage through the various isolation valves has been attributed to causes