Public Participation

The CAP meeting is open to the interested public, but limited to the space available. Persons wishing to attend should notify the CAP Facilitator at least two days before the meeting. Any member of the public may file a written statement with the CAP Facilitator before the meeting. Minutes of the meeting will be available on request.

Dated: March 8, 1995.

Doris Meissner,

Commissioner, Immigration and Naturalization Service.

[FR Doc. 95–6213 Filed 3–13–95; 8:45 am]

BILLING CODE 4410–10–M

NATIONAL AERONAUTICS AND SPACE ADMINISTRATION

[Notice (95–023)]

Intent To Grant an Exclusive Patent License

AGENCY: National Aeronautics and Space Administration.

ACTION: Notice of intent to grant a patent license.

SUMMARY: NASA hereby gives notice of its intent to negotiate with KVH Industries, Inc., of Middletown, Rhode Island, an exclusive, royalty-bearing revocable license to practice the invention described in U.S. Patent Application Serial Number 07/999/794, filed November 30, 1992, entitled ''Satellite-Tracking Millimeter Wave Reflector Antenna System for Mobile Satellite Tracking,'' which will issue on March 14, 1995, to the United States of America as represented by the Administrator of the National Aeronautics and Space Administration. The proposed license agreement will contain appropriate terms, limitations, and conditions to be negotiated in accordance with the regulations governing the licensing of government-owned inventions as described in 37 CFR part 404. NASA will negotiate the final terms and conditions and grant the exclusive license, unless within 60 days of the date of this Notice, the NASA Director of Patent Licensing receives written objections to the grant of an exclusive license, together with any supporting documentation. The NASA Director of Patent Licensing will review all written responses to the notice and then recommend to the Associate General Counsel (Intellectual Property) whether or not to grant the exclusive license.

DATES: Comments to the notice must be received by May 15, 1995.

ADDRESSES: National Aeronautics and Space Administration, Code GP, Washington, DC 20546.

FOR FURTHER INFORMATION CONTACT: Mr. Harry Lupuloff, NASA, Director of Patent Licensing, (202) 358–2041.

Dated: March 6, 1995.

Edward A. Frankle,

General Counsel.

[FR Doc. 95–6233 Filed 3–13–95; 8:45 am]

BILLING CODE 7510–01–M

NUCLEAR REGULATORY COMMISSION

Organization of Agreement State Managers' Meeting

AGENCY: U.S. Nuclear Regulatory Commission.

ACTION: Notice of meeting.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) staff plans to hold a public meeting for managers of the Organization of Agreement States. Agreement States are States which have assumed regulatory authority over certain radioactive materials. The purpose of the meeting is to discuss Agreement State Program issues with Agreement State managers and other interested parties. Topics for discussion will include, among others: Status of NRC Agreement States Program Improvements; Open Discussion of Agreement State Issues; Integrated Materials Performance Evaluation Program; Event Reporting and Data Gathering; NRC Materials Licensing Business Process Redesign Project; and Licensee Wrongdoing Awareness Workshop.

DATES: The meeting will be held on April 5–6, 1995 from 8:30 a.m. to 5:00 p.m. each day.

ADDRESSES: The meeting will be held at NRC's Two White Flint North Auditorium, located at 11545 Rockville Pike, Rockville, Maryland.

FOR FURTHER INFORMATION CONTACT: Rosetta Virgilio, Office of State Programs, Mail Stop OWFN–3–D–23, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Telephone 301/415–2307.

CONDUCT OF THE MEETING: The meeting will be conducted in a manner that will expedite the orderly conduct of business. A transcript of the meeting will be available for inspection, and copying for a fee, at the NRC Public Document Room, 2120 L Street N.W. (Lower Level), Washington, D.C. 20555 on or about June 5, 1995.

The following procedures apply to public attendance at the meeting:

1. Questions or statements will be entertained on a first-come, first-served basis.

2. Seating will be on a first-come, first-served basis.

Dated at Rockville, Maryland this 6th day of March, 1995.

For the U.S. Nuclear Regulatory Commission.

Richard L. Bangart,

Director, Office of State Programs.

[FR Doc. 95–6208 Filed 3–13–95; 8:45 am]

BILLING CODE 7590–01–M

[Docket No. 50–213]

Connecticut Yankee Atomic Power Co.; Notice of Issuance of Amendment to Facility Operating License

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment No. 184 to Facility Operating License No. DPR–61 issued to the Connecticut Yankee Atomic Power Company (the licensee), which revised the Technical specifications for operation of the Haddam Neck Plant located in Middlesex County, Connecticut. The amendment is effective as of the date of issuance to be implemented within 30 days of issuance.

The amendment revises Technical Specification (TS) 3/4.4.10, ''Structural Integrity,'' surveillance requirement 4.4.10. In particular, the change will add a footnote to the TS for an extension for one cycle of the ultrasonic volumetric inspection for the areas of higher stress concentration for RCPs 1 and 2.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity for Hearing in connection with this action was published in the **Federal Register** on February 8, 1994 (59 FR 5787). No request for a hearing or petition for leave to intervene was filed following the notice.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of the amendment will not have a significant effect on the quality

of the human environment (60 FR 11124).

For further details with respect to the action see (1) the application for amendment dated December 22, 1993, (2) Amendment No. 184 to License No. DPR–61, (3) the Commission's related Safety Evaluation, and (4) the Commission's Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Russell Library, 123 Broad Street, Middletown, CT 06457.

Dated at Rockville, Maryland, this 6th day of March 1995.

For the Nuclear Regulatory Commission.

Alan B. Wang,

Project Manager, Project Directorate I–4, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

[FR Doc. 95–6209 Filed 3–13–95; 8:45 am]

BILLING CODE 7590–01–M

[Docket No. 50–413]

Duke Power Company, et al. (Catawba Nuclear Station, Unit No. 1); Exemption

I

The Duke Power Company, et al. (DPC or the licensee) is the holder of Facility Operating License No. NPF–35, which authorizes operation of the Catawba Nuclear Station, Unit No. 1 (the facility), at a steady-state reactor power level not in excess of 3411 megawatts thermal. The facility is a pressurized water reactor located at the licensee's site in York County, South Carolina. The license provides, among other things, that the Catawba Nuclear Station is subject to all rules, regulations, and Orders of the U.S. Nuclear Regulatory Commission (the Commission or NRC) now or hereafter in effect.

II

Section III.D.1.(a) of Appendix J to 10 CFR Part 50 requires the performance of three Type A containment integrated leakage rate tests (ILRTs) at approximately equal intervals during each 10-year service period of the primary containment. The third test of each set shall be conducted when the plant is shut down for a 10-year inservice inspection of the primary containment.

III

By letters dated October 18, 1994, and February 7, 1995, the licensee requested temporary relief from the requirement to perform a set of three Type A tests at approximately equal intervals during each 10-year service period of the primary containment. The requested exemption would permit a one-time interval extension of the third Type A test by approximately 16 months (from the 1995 refueling outage, which began on February 11, 1995, to the end-of-cycle 9 (EOC–9) refueling outage, currently scheduled for June 1996) and would permit the third Type A test of the second 10-year inservice inspection period to not correspond with the end of the current inservice inspection interval.

The licensee's request concluded that the proposed change, a one-time extension of the interval between the second and third ILRTs at Catawba Unit 1, is justified for the following reasons:

The previous testing history at Catawba Unit 1 provides substantial justification for the proposed test interval extension. In each of the two previous periodic ILRTs at Catawba Unit 1, the as-found leakage was less than or equal to 22.5% of the allowable leakage, thereby demonstrating that Catawba Unit 1 is a low-leakage containment. There are no mechanisms which would adversely affect the structural integrity of the containment, or that would be a factor in extending the test interval by 20 months. However, as a preventative maintenance measure, a containment civil inspection, currently required by Appendix J prior to a Type A test, will be performed during EOC–8 to verify that no structural degradation exists. Any additional risk created by the longer interval between ILRTs is considered to be negligible, primarily because Type B and C testing will continue unchanged.

Additionally, the licensee stated that its exemption request meets the requirements of 10 CFR 50.12, paragraphs (a)(1) and (a)(2)(ii), for the following reasons:

In order to justify the granting of an exemption to the requirements of 10 CFR Part 50, paragraph 50.12(a)(1) requires that the licensee show that the proposed exemption will not pose an undue risk to the public. That this proposed change will not pose an undue risk is demonstrated by the analysis presented in draft NUREG–1493, which concludes that an increase in the test interval to once every 20 years would ''lead to an imperceptible increase in risk.'' The analyses in draft NUREG–1493 are considered to be specifically applicable to Catawba because: (1) The requested exemption would result in a one-time increase in the test interval to 5 years, not 20; (2) the population density around Catawba is less than that used in the study (329 people per square mile, vs. 340 used in the study; (3) no ILRT at Catawba has jailed; (4) the core inventory used in the study was represented by a 3412 Mwt PWR [pressurized water reactor]. Catawba is a 3411 Mwt PWR. Other factors which lead to the conclusion that the proposed change will not pose an undue risk include the fact that local leak rate testing, which identifies 97% of leakage in excess of prescribed limits, will remain in place at its current test frequency; the detailed, proceduralize containment civil inspection which is normally performed in conjunction with an ILRT will be performed in place of the scheduled ILRT, to identify potential structural deteriorations; and the historical leak-tightness of the containment structure, as evidenced by two successive ILRTs in which the as-found leakage did not exceed 22.5% of the allowable leakage rate.

A comparison was made between the risk analysis presented in draft NUREG–1493 and a probabilistic risk assessment performed for Catawba Nuclear Station. While the quantitative results of the NUREG are not directly applicable to plants not used in the study, similar conclusions can be made concerning Catawba. NUREG–1493 indicates that reactor accident risks are dominated by accident sequences that result in failure or bypass of the containment. This conclusion is also valid for Catawba. Considering only the Catawba accident sequences that do not result in containment failure, containment leakage contributes approximately 0.08 to 0.09 percent to off-site risk (whole-body person-rem, thyroid nodules, and latent fatalities). NUREG–1493 indicated that containment leakage contributed from 0.02 to 0.10 percent to latent cancer risk. The comparison between the analysis of NUREG–1493 and the Catawba PRA concludes that increases in containment leakage at Catawba are expected to produce increases in accident risk similar to the results in NUREG–1493.

Special circumstances, as defined in 10 CFR [50].12(a)(2)(ii), are present in that the requirement to perform the third ILRT during the ISI outage is not necessary to achieve the underlying purpose of the rule. The purposes of the rule, as stated in Section I of Appendix J, are to ensure that: (a) Leakage through the primary reactor containment and systems and components penetrating containment shall not exceed allowable values, and (b) periodic surveillance of reactor containment penetrations and isolation valves is performed so that proper maintenance and repairs are made. One of the significant factors in assuring that the proposed exemption will not pose an undue risk to the public, as noted above, is the local leak rate testing (LLRT) which is performed. That the LLRT program at Catawba provides an effective mechanism for maintaining containment integrity is perhaps best demonstrated by the fact that the most recent ILRT at Catawba Unit 1 was performed at the front end of the refueling outage; before any repairs or adjustments were made to valves or penetrations. Nevertheless, the as-found leakage did not exceed 22.5% of the allowable leakage rate. The fact that no leakage paths were identified by an ILRT, and that the ILRT met the acceptance criteria with significant margin confirms the results of the Type B and C testing.

The frequency and scope of the Type B and C LLRT program are not being changed by this exemption request. The LLRT program