

30 and Gable Road, St. Helens, Oregon. The meeting will begin at 7:30 pm and will last approximately two hours. The purpose of the meeting is to provide the NRC and the ODOE an opportunity to explain their respective decommissioning review processes and to provide interested members of the public an opportunity to ask questions and provide comments on the decommissioning of the Trojan plant.

Copies of the Decommissioning Plan and Environmental Report are available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC 20555, and at the Local Public Document Room located on the fifth floor of the Branford Price Millar Library, Portland State University, 934 S.W. Harrison Street, P.O. Box 1151, Portland, Oregon 97207.

Dated at Rockville, Maryland, this 28th day of February 1995.

For the Nuclear Regulatory Commission.

Seymour H. Weiss,

Director, Non-Power Reactors and Decommissioning Project Directorate, Division of Project Support, Office of Nuclear Reactor Regulation.

[FR Doc. 95-5613 Filed 3-7-95; 8:45 am]

BILLING CODE 7590-01-M

Regulatory Guide; Issuance, Availability

The Nuclear Regulatory Commission has issued a new guide in its Regulatory Guide Series. This series has been developed to describe and make available to the public such information as methods acceptable to the NRC staff for implementing specific parts of the Commission's regulations, techniques used by the staff in evaluating specific problems or postulated accidents, and data needed by the staff in its review of applications for permits and licenses.

Regulatory Guide 6.9, "Establishing Quality Assurance Programs for the Manufacture and Distribution of Sealed Sources and Devices Containing Byproduct Material," provides guidance acceptable to the NRC staff on the essential elements needed to develop, establish, and maintain a quality assurance program for the manufacture and distribution of sealed sources and devices.

Comments and suggestions in connection with items for inclusion in guides currently being developed or improvements in all published guides are encouraged at any time. Written comments may be submitted to the Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of

Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555.

Regulatory guides are available for inspection at the Commission's Public Document Room, 2120 L Street NW., Washington, DC. Copies of issued guides may be purchased from the Government Printing Office at the current GPO price. Information on current GPO prices may be obtained by contacting the Superintendent of Documents, U.S. Government Printing Office, Post Office Box 37082, Washington, DC 20013-7082, telephone (202) 512-2249. Issued guides may also be purchased from the National Technical Information Service on a standing order basis. Details on this service may be obtained by writing NTIS, 5285 Port Royal Road, Springfield, VA 22161.

(5 U.S.C. 552(a))

Dated at Rockville, Maryland, this 13th day of February 1995.

For the Nuclear Regulatory Commission.

Eric S. Beckjord,

Director, Office of Nuclear Regulatory Research.

[FR Doc. 95-5609 Filed 3-7-95; 8:45 am]

BILLING CODE 7590-01-M

[Docket No. 50-318]

Baltimore Gas and Electric Co.; Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR-69 issued to Baltimore Gas and Electric Company (the licensee) for operation of the Calvert Cliffs Nuclear Power Plant, Unit No. 2, located in Calvert County, Maryland.

The proposed amendment would revise the Calvert Cliffs, Unit No. 2, Technical Specifications (TSs). Specifically, TS 4.G.1.2 would reference 10 CFR part 50, Appendix J directly, and any approved exemptions to the Type A testing frequently requirements, rather than paraphrase the regulation. The proposed wording is consistent with that used in NUREG-1432, "Standard Technical Specifications—Combustion Engineering Plants," dated September 1992.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Would not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change revises Technical Specification 4.6.1.2.a to reference the testing frequency requirements of 10 CFR part 50, Appendix J, and to state that NRC-approved exemptions to the applicable regulatory requirements are permitted. The current Technical Specification 4.6.1.2.a paraphrases the requirements of Appendix J, paragraph III.D.1.(a) and necessitates a change to the Technical Specifications should the Appendix J language change or an exemption be granted. The proposed administrative revision simply deletes the paraphrased language and directly references Appendix J and any approved exemptions. No new requirements are added, nor are any existing requirements deleted. Any specific exemptions from the requirements of Appendix J, paragraph III.D.1.(a) will continue to require a submittal from Baltimore Gas and Electric Company under 10 CFR 50.12 and subsequent review and approval by the NRC prior to implementation.

The proposed change will provide a one-time exemption from the 10 CFR part 50, Appendix J, paragraph III.D.1.(a) leak rate test schedule requirement. This change will allow for a one-time interval between subsequent Type A test of approximately 72 months. It will also extend the second ten-year Type A testing service period to 12 years to coincide with the inservice inspection interval.

No physical or operational changes to the structure, plant systems or components would be made as a result of the proposed change. Furthermore, leak rate testing is not an initiating event in any accident, therefore this proposed change does not involve a significant increase in the probability of any accident previously evaluated.

Type A tests are capable of detecting containment leaks through containment penetrations and through the containment liner. The history at Calvert Cliffs Unit 2 demonstrates that Type B and C Local Leak Rate Tests (LLRTs) have consistently detected leakage through penetrations. With the exception of the first periodic Unit 2 Type A test in 1979, which failed and was promptly corrected, Type A tests have not