

(The meeting will be closed to the public at this point.)

The remainder of the proposed meeting will be given to the consideration of specific applications (closed to the public for the reasons stated above).

Further information about this meeting can be obtained from Mr. David C. Fisher, Advisory Committee Management Officer, Washington, DC 20506, or call area code (202) 606-8322, TDD (202) 606-8282. Advance notice of any special needs or accommodations is appreciated.

David C. Fisher,

Advisory Committee Management Officer.

[FR Doc. 95-3007 Filed 2-6-95; 8:45 am]

BILLING CODE 7536-01-M

NATIONAL SCIENCE FOUNDATION

DOE/NSF Nuclear Science Advisory Committee; Notice of Meeting

In accordance with the Federal Advisory Committee Act (Pub. L. 92-463, as amended), the National Science Foundation announces the following meeting.

Name: DOE/NSF Nuclear Science Advisory Committee.

Date and Time: February 24, 1995 from 8:30 a.m. to 7:00 p.m., February 25, 1995 from 8:30 a.m. to 4:00 p.m.

Place: Arlington Renaissance Hotel, Gallery II, 950 North Stafford Street, Arlington, VA 22203.

Type of Meeting: Open.

Contact Person: John W. Lightbody, Program Director for Nuclear Physics, National Science Foundation, 4201 Wilson Blvd., Arlington, VA 22230, Telephone: (703) 306-1890.

Minutes: May be obtained from the contact person listed above.

Purpose of Meeting: To advise the National Science Foundation and the Department of Energy on scientific priorities within the field of basic nuclear science research.

Agenda

February 24, 1995

- Discussion of Budgets and Status of DOE and NSF Nuclear Physics Programs (D. Hendrie, DOE; J. Lightbody, NSF)

- Presentation of Preliminary Subcommittee Report regarding Additional Capital Equipment for the RHIC Facility (C. Gelbke)

- Reports of Town Meetings of the Division of Nuclear Physics of the American Physical Society (by conveners)

February 25, 1995

- Discussion of Town Meeting Reports
- Progress Reports of the Long Range Plan Working Groups (LRPWG)

- Discussion of process and plans for full LRPWG Meeting

- Public Comment (*)

(*) Persons wishing to speak should make arrangements through the Contact Person identified above.

Dated: February 2, 1995.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 95-2947 Filed 2-6-95; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-352 and 50-353]

Philadelphia Electric Company; Limerick Generating Station, Units 1 and 2; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering two actions: (1) Issuance of an exemption from the requirements of 10 CFR 50, Appendix J, and (2) an amendment to Facility Operating License Nos. NPF-39 and NPF-85, issued to Philadelphia Electric Company (the licensee), for operation of the Limerick Generating Station (LGS), Units 1 and 2, located in Montgomery County, Pennsylvania.

Environmental Assessment

Identification of the Proposed Action

The proposed action would grant (1) an exemption from 10 CFR Part 50, Appendix J, Sections II.H.4, III.C.2, and III.C.3, and (2) an amendment to change the Technical Specifications (TS) for the Limerick Generating Station (LGS), Units 1 and 2, in conjunction with the removal of the main steam isolation valve (MSIV) leakage control system (LCS) and the proposed use of an alternate leakage pathway.

10 CFR Part 50, Appendix J, Sections II.H.4 and III.C.2 require leak rate testing of MSIVs at the calculated peak containment pressure related to the design basis accident, and Section III.C.3 requires that the measured MSIV leak rates be included in the combined local leak rate test results. The proposed deletion of the MSIV LCS and proposed use of an alternate leakage pathway affects the description of an existing exemption (NUREG-0991, and its Supplement 3), which allows the leak rate testing of the MSIVs at a reduced pressure and allows exclusion of the measured MSIV leakage from the combined local leak rate test results.

The proposed TS amendment would permit an increase in the allowable MSIV leakage rate from 11.5 standard cubic feet per hour (scfh) to 100 scfh for any one MSIV and a combined maximum pathway leakage rate of 200 scfh for all four main steam lines, and would delete TS requirements for the currently installed MSIV LCS, because

the proposed system removal makes the TS inapplicable.

The proposed action for the TS amendments is in accordance with the licensee's application for amendment dated January 14, 1994, as supplemented by letters dated August 1, October 25, December 13, and December 22, 1994; and the proposed action for the exemption is in accordance with the letter dated December 22, 1994.

The Need for the Proposed Action

The proposed exemption is similar to the current exemption from 10 CFR Part 50, Appendix J, Sections II.H.4 and III.C.2. The exemption is needed since the design of the MSIVs is such that testing in the reverse direction tends to unseat the valve and would result in a meaningless test. The total observed MSIV leak rate resulting from a leakage test where two MSIVs on one steam line are tested utilizing a reduced pressure (22 psig) will continue to be assigned to the penetration. The proposed exemption is also similar to the current exemption from 10 CFR Part 50, Appendix J, Section III.C.3. The licensee proposes that the MSIV leakage rate will continue to be accounted for separately in the radiological site analysis in accordance with the existing exemption. However, the existing exemption from 10 CFR Part 50, Appendix J, Section III.C.3 will not be applicable when the MSIV LCS is replaced with an Alternate Treatment Path (ATP) (main steam lines and condenser).

The proposed action regarding the TS amendment will reduce the need for repairs of the MSIVs, resolve concerns associated with the current LCS performance capability at high MSIV leakage rates, and provide an effective method for dealing with a potential MSIV leakage during a postulated loss-of-coolant accident (LOCA). Many boiling water reactors (BWRs) have difficulty meeting their MSIV leakage rate limits. Extensive repair, rework, and retesting efforts have negative effects on the outage costs and schedules, as well as significant impact on the licensee's as low as is reasonably achievable (ALARA) radiological exposure programs. The alternatives proposed by the licensee to deal with MSIV leakage make use of components (main steam lines and condenser) that are expected to remain intact and serviceable following a design basis LOCA.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed actions related to the granting of an exemption