

TABLE 5—CONVEYANCE ACTIVITY LIMITS FOR LSA MATERIAL AND SCO—Continued

Nature of material	Activity limit for conveyances
3. LSA-II and LSA-III; Combustible solids and all liquids and gases.	100 A ₂
4. SCO	100 A ₂

TABLE 6—INDUSTRIAL PACKAGE INTEGRITY REQUIREMENTS FOR LSA MATERIAL AND SCO

Contents	Industrial packaging type	
	Exclusive use shipment	Non exclusive use shipment
1. LSA-I:		
Solid	IP-1 ...	IP-1
Liquid	IP-1 ...	IP-2
2. LSA-II:		
Solid	IP-2 ...	IP-2
Liquid and gas	IP-2 ...	IP-3
3. LSA-III	IP-2 ...	IP-3
4. SCO-I	IP-1 ...	IP-1
5. SCO-II	IP-2 ...	IP-2

[69 FR 3676, Jan. 26, 2004; 69 FR 55118, Sept. 13, 2004; 69 FR 58843, Oct. 1, 2004; 70 FR 56098, Sept. 23, 2005]

§ 173.428 Empty Class 7 (radioactive) materials packaging.

A packaging which previously contained Class 7 (radioactive) materials and has been emptied of contents as far as practical, is excepted from the shipping paper and marking (except for the UN identification number marking requirement described in § 173.422(a)) requirements of this subchapter, provided that—

(a) The packaging meets the requirements of § 173.421(a) (2), (3), and (5) of this subpart;

(b) The packaging is in unimpaired condition and is securely closed so that there will be no leakage of Class 7 (radioactive) material under conditions normally incident to transportation;

(c) The outer surface of any uranium or thorium in its structure is covered with an inactive sheath made of metal or some other substantial material;

(d) Internal contamination does not exceed 100 times the limits in § 173.443(a);

(e) Any labels previously applied in conformance with subpart E of part 172

of this subchapter are removed, obliterated, or covered and the “Empty” label prescribed in § 172.450 of this subchapter is affixed to the packaging; and

(f) The packaging is prepared for shipment as specified in § 173.422.

[Amdt. 173-244, 60 FR 50307, Sept. 28, 1995, as amended by Amdt. 173-244, 61 FR 20752, May 8, 1996; 64 FR 51919, Sept. 27, 1999; 69 FR 3677, Jan. 26, 2004]

§ 173.431 Activity limits for Type A and Type B packages.

(a) Except for LSA material and SCO, a Type A package may not contain a quantity of Class 7 (radioactive) materials greater than A₁ for special form Class 7 (radioactive) material or A₂ for normal form Class 7 (radioactive) material as listed in § 173.435, or, for Class 7 (radioactive) materials not listed in § 173.435, as determined in accordance with § 173.433.

(b) The limits on activity contained in a Type B(U) or Type B(M) package are those prescribed in §§ 173.416 and 173.417, or in the applicable approval certificate under §§ 173.471, 173.472 or 173.473.

[Amdt. 173-244, 60 FR 50307, Sept. 28, 1995, as amended at 69 FR 3677, Jan. 26, 2004]

§ 173.433 Requirements for determining basic radionuclide values, and for the listing of radionuclides on shipping papers and labels.

(a) For individual radionuclides listed in the table in § 173.435 and § 173.436:

(1) A₁ and A₂ values are given in the table in § 173.435; and

(2) Activity concentration exemption values and consignment activity exemption values are given in the table in § 173.436.

(b) For individual radionuclides which are not listed in the tables in § 173.435 or § 173.436:

(1) the radionuclide values in Tables 7 or 8 of this section may be used; or

(2) other basic radionuclide values may be used provided they are first approved by the Associate Administrator or, for international transport, multilateral approval is obtained from the pertinent Competent Authorities.

(c) In calculating A₁ or A₂ values for a radionuclide not listed in the table in § 173.435:

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(1) Where the chemical form of each radionuclide is known, it is permissible to use the A_2 value related to its solubility class as recommended by the International Commission on Radiological Protection, if the chemical forms under both normal and accident conditions of transport are taken into consideration.

(2) A single radioactive decay chain in which the radionuclides are present in their naturally-occurring proportions, and in which no daughter nuclide has a half life either longer than 10 days or longer than that of the parent nuclide, will be considered as a single radionuclide, and the activity to be taken into account and the A_1 or A_2 value to be applied will be those corresponding to the parent nuclide of that chain. Otherwise, the parent and daughter nuclides will be considered as a mixture of different nuclides.

(d) Mixtures of radionuclides whose identities and respective activities are known must conform to the following conditions:

(1) For special form Class 7 (radioactive) material, the activity which may be transported in a Type A package must satisfy:

$$\sum_i \frac{B(i)}{A_1(i)} \leq 1$$

Where:

$B(i)$ is the activity of radionuclide i in special form; and
 $A_1(i)$ is the A_1 value for radionuclide i .

(2) For normal form Class 7 (radioactive) material, the activity which may be transported in a Type A package must satisfy:

$$\sum_j \frac{C(j)}{A_2(j)} \leq 1$$

Where:

$f(i)$ is the fraction of activity concentration of nuclide i in the mixture; and $[A](i)$ is

Where:

$C(j)$ is the activity of radionuclide j in normal form; and

$A_2(j)$ is the A_2 value for radionuclide j .

(3) If the package contains both special and normal form Class 7 (radioactive) material, the activity which may be transported in a Type A package must satisfy:

$$\sum_i \frac{B(i)}{A_1(i)} + \sum_j \frac{C(j)}{A_2(j)} \leq 1$$

Where:

The symbols are defined as in paragraphs (d)(2) and (d)(3) of this section.

(4) Alternatively, the A_1 value for a mixture of special form material may be determined as follows:

$$A_1 \text{ for mixture} = \frac{1}{\sum_i \frac{f(i)}{A_1(i)}}$$

Where:

$f(i)$ is the fraction of activity for radionuclide i in the mixture; and
 $A_1(i)$ is the appropriate A_1 value for radionuclide i .

(5) Alternatively, the A_2 value for mixtures of normal form material may be determined as follows:

$$A_2 \text{ for mixture} = \frac{1}{\sum_i \frac{f(i)}{A_2(i)}}$$

Where:

$f(i)$ is the fraction of activity for normal form radionuclide i in the mixture; and
 $A_2(i)$ is the appropriate A_2 value for radionuclide i .

(6) The exempt activity concentration for mixtures of nuclides may be determined as follows:

$$\text{Exempt activity concentration limit for mixture} = \frac{1}{\sum_i \frac{f(i)}{[A](i)}}$$

the activity concentration for exempt material containing nuclide i .

(7) The activity limit for an exempt consignment for mixtures of nuclides may be determined as follows:

$$\text{Exempt consignment activity limit for mixture} = \frac{1}{\sum_i \frac{f(i)}{A(i)}}$$

Where:

f(i) is the fraction of activity of nuclide i in the mixture; and

A(i) is the activity limit for exempt consignments for nuclide i.

(e) When the identity of each nuclide is known but the individual activities of some of the radionuclides are not known, the radionuclides may be grouped and the lowest A₁ or A₂ value, as appropriate, for the radionuclides in each group may be used in applying the formulas in paragraphs (d)(1) through (d)(5) of this section. Groups may be based on the total alpha activity and the total beta/gamma activity when these are known, using the lowest A₁ or A₂ values for the alpha emitters or beta/gamma emitters, respectively.

(f) When the identity of each nuclide is known but the individual activities of some of the radionuclides are not known, the radionuclides may be grouped and the lowest [A] (activity concentration for exempt material) or A (activity limit for exempt consignment) value, as appropriate, for the radionuclides in each group may be used in applying the formulas in para-

graphs (d)(6) and (d)(7) of this section. Groups may be based on the total alpha activity and the total beta/gamma activity when these are known, using the lowest [A] or A values for the alpha emitters or beta/gamma emitters, respectively.

(g) *Shipping papers and labeling.* For mixtures of radionuclides, the radionuclides (n) that must be shown on shipping papers and labels in accordance with §§172.203 and 172.403 of this subchapter, respectively, must be determined on the basis of the following formula:

$$\sum_{i=1}^n \frac{a_{(i)}}{A_{(i)}} \geq 0.95 \sum_{i=1}^{n+m} \frac{a_{(i)}}{A_{(i)}}$$

Where:

n + m represents all the radionuclides in the mixture;

m are the radionuclides that do not need to be considered;

a_(i) is the activity of radionuclide i in the mixture; and

A_(i) is the A₁ or A₂ value, as appropriate for radionuclide i.

(h) Tables 7 and 8 are as follows:

TABLE 7—GENERAL VALUES FOR A₁ AND A₂

Radioactive contents	A ₁		A ₂	
	(TBq)	(Ci)	(TBq)	(Ci)
1. Only beta or gamma emitting nuclides are known to be present	1 × 10 ⁻¹	2.7 × 10 ⁰	2 × 10 ⁻²	5.4 × 10 ⁻¹
2. Only alpha emitting nuclides are known to be present	2 × 10 ⁻¹	5.4 × 10 ⁰	9 × 10 ⁻⁵	2.4 × 10 ⁻³
3. No relevant data are available	1 × 10 ⁻³	2.7 × 10 ⁻²	9 × 10 ⁻⁵	2.4 × 10 ⁻³

TABLE 8—GENERAL EXEMPTION VALUES

Radioactive contents	Activity concentration for exempt material		Activity limits for exempt consignments	
	(Bq/g)	(Ci/g)	(Bq)	(Ci)
1. Only beta or gamma emitting nuclides are known to be present	1 × 10 ¹	2.7 × 10 ⁻¹⁰	1 × 10 ⁴	2.7 × 10 ⁻⁷
2. Only alpha emitting nuclides are known to be present	1 × 10 ⁻¹	2.7 × 10 ⁻¹²	1 × 10 ³	2.7 × 10 ⁻⁸
3. No relevant data are available	1 × 10 ⁻¹	2.7 × 10 ⁻¹²	1 × 10 ³	2.7 × 10 ⁻⁸

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[69 FR 3677, Jan. 26, 2004; 69 FR 55119, Sept. 13, 2004]

§ 173.434 Activity-mass relationships for uranium and natural thorium.

The table of activity-mass relationships for uranium and natural thorium are as follows:

Thorium and uranium enrichment ¹ (Wt% ²³⁵ U present)	Specific activity			
	TBq/gram	Grams/Tbq	Ci/gram	Grams/Ci
0.45 (depleted)	1.9×10 ⁻⁸	5.4×10 ⁷	5.0×10 ⁻⁷	2.0×10 ⁶
0.72 (natural)	2.6×10 ⁻⁸	3.8×10 ⁷	7.1×10 ⁻⁷	1.4×10 ⁶
1.0	2.8×10 ⁻⁸	3.6×10 ⁷	7.6×10 ⁻⁷	1.3×10 ⁶
1.5	3.7×10 ⁻⁸	2.7×10 ⁷	1.0×10 ⁻⁶	1.0×10 ⁶
5.0	1.0×10 ⁻⁷	1.0×10 ⁷	2.7×10 ⁻⁶	3.7×10 ⁵
10.0	1.8×10 ⁻⁷	5.6×10 ⁶	4.8×10 ⁻⁶	2.1×10 ⁵
20.0	3.7×10 ⁻⁷	2.7×10 ⁶	1.0×10 ⁻⁵	1.0×10 ⁵
35.0	7.4×10 ⁻⁷	1.4×10 ⁶	2.0×10 ⁻⁵	5.0×10 ⁴
50.0	9.3×10 ⁻⁷	1.1×10 ⁶	2.5×10 ⁻⁵	4.0×10 ⁴
90.0	2.1×10 ⁻⁶	4.7×10 ⁵	5.8×10 ⁻⁵	1.7×10 ⁴
93.0	2.6×10 ⁻⁶	3.9×10 ⁵	7.0×10 ⁻⁵	1.4×10 ⁴
95.0	3.4×10 ⁻⁶	3.0×10 ⁵	9.1×10 ⁻⁵	1.1×10 ⁴
Natural thorium	8.1×10 ⁻⁹	1.2×10 ⁸	2.2×10 ⁻⁷	4.6×10 ⁶

¹ The figures for uranium include representative values for the activity of uranium-234 which is concentrated during the enrichment process. The activity for thorium includes the equilibrium concentration of thorium-228.

[Amdt. 173–244, 60 FR 50307, Sept. 28, 1995, as amended by 63 FR 52849, Oct. 1, 1998]

§ 173.435 Table of A₁ and A₂ values for radionuclides.

The table of A₁ and A₂ values for radionuclides is as follows:

Symbol of radionuclide	Element and atomic number	A ₁ (TBq)	A ₁ (Ci) ^b	A ₂ (TBq)	A ₂ (Ci) ^b	Specific activity	
						(TBq/g)	(Ci/g)
Ac-225 (a)	Actinium (89)	8.0×10 ⁻¹	2.2×10 ¹	6.0×10 ⁻³	1.6×10 ⁻¹	2.1×10 ³	5.8×10 ⁴
Ac-227 (a)	9.0×10 ⁻¹	2.4×10 ¹	9.0×10 ⁻⁵	2.4×10 ⁻³	2.7	7.2×10 ¹
Ac-228	6.0×10 ⁻¹	1.6×10 ¹	5.0×10 ⁻¹	1.4×10 ¹	8.4×10 ⁴	2.2×10 ⁶
Ag-105	Silver (47)	2.0	5.4×10 ¹	2.0	5.4×10 ¹	1.1×10 ³	3.0×10 ⁴
Ag-108m (a)	7.0×10 ⁻¹	1.9×10 ¹	7.0×10 ⁻¹	1.9×10 ¹	9.7×10 ⁻¹	2.6×10 ¹
Ag-110m (a)	4.0×10 ⁻¹	1.1×10 ¹	4.0×10 ⁻¹	1.1×10 ¹	1.8×10 ²	4.7×10 ³
Ag-111	2.0	5.4×10 ¹	6.0×10 ⁻¹	1.6×10 ¹	5.8×10 ³	1.6×10 ⁵
Al-26	Aluminum (13)	1.0×10 ⁻¹	2.7	1.0×10 ⁻¹	2.7	7.0×10 ⁻⁴	1.9×10 ⁻²
Am-241	Americium (95)	1.0×10 ¹	2.7×10 ²	1.0×10 ⁻³	2.7×10 ⁻²	1.3×10 ⁻¹	3.4
Am-242m (a)	1.0×10 ¹	2.7×10 ²	1.0×10 ⁻³	2.7×10 ⁻²	3.6×10 ⁻¹	1.0×10 ¹
Am-243 (a)	5.0	1.4×10 ²	1.0×10 ⁻³	2.7×10 ⁻²	7.4×10 ⁻³	2.0×10 ⁻¹
Ar-37	Argon (18)	4.0×10 ¹	1.1×10 ³	4.0×10 ¹	1.1×10 ³	3.7×10 ³	9.9×10 ⁴
Ar-39	4.0×10 ¹	1.1×10 ³	2.0×10 ¹	5.4×10 ²	1.3	3.4×10 ¹
Ar-41	3.0×10 ⁻¹	8.1	3.0×10 ⁻¹	8.1	1.5×10 ⁶	4.2×10 ⁷
As-72	Arsenic (33)	3.0×10 ⁻¹	8.1	3.0×10 ⁻¹	8.1	6.2×10 ⁴	1.7×10 ⁶
As-73	4.0×10 ¹	1.1×10 ³	4.0×10 ¹	1.1×10 ³	8.2×10 ²	2.2×10 ⁴
As-74	1.0	2.7×10 ¹	9.0×10 ⁻¹	2.4×10 ¹	3.7×10 ³	9.9×10 ⁴
As-76	3.0×10 ⁻¹	8.1	3.0×10 ⁻¹	8.1	5.8×10 ⁴	1.6×10 ⁶
As-77	2.0×10 ¹	5.4×10 ²	7.0×10 ⁻¹	1.9×10 ¹	3.9×10 ⁴	1.0×10 ⁶
At-211 (a)	Astatine (85)	2.0×10 ¹	5.4×10 ²	5.0×10 ⁻¹	1.4×10 ¹	7.6×10 ⁴	2.1×10 ⁶
Au-193	Gold (79)	7.0	1.9×10 ²	2.0	5.4×10 ¹	3.4×10 ⁴	9.2×10 ⁵
Au-194	1.0	2.7×10 ¹	1.0	2.7×10 ¹	1.5×10 ⁴	4.1×10 ⁵
Au-195	1.0×10 ¹	2.7×10 ²	6.0	1.6×10 ²	1.4×10 ²	3.7×10 ³
Au-198	1.0	2.7×10 ¹	6.0×10 ⁻¹	1.6×10 ¹	9.0×10 ³	2.4×10 ⁵
Au-199	1.0×10 ¹	2.7×10 ²	6.0×10 ⁻¹	1.6×10 ¹	7.7×10 ³	2.1×10 ⁵
Ba-131 (a)	Barium (56)	2.0	5.4×10 ¹	2.0	5.4×10 ¹	3.1×10 ³	8.4×10 ⁴
Ba-133	3.0	8.1×10 ¹	3.0	8.1×10 ¹	9.4	2.6×10 ²
Ba-133m	2.0×10 ¹	5.4×10 ²	6.0×10 ⁻¹	1.6×10 ¹	2.2×10 ⁴	6.1×10 ⁵
Ba-140 (a)	5.0×10 ⁻¹	1.4×10 ¹	3.0×10 ⁻¹	8.1	2.7×10 ³	7.3×10 ⁴
Be-7	Beryllium (4)	2.0×10 ¹	5.4×10 ²	2.0×10 ¹	5.4×10 ²	1.3×10 ⁴	3.5×10 ⁵
Be-10	4.0×10 ¹	1.1×10 ³	6.0×10 ⁻¹	1.6×10 ¹	8.3×10 ⁻⁴	2.2×10 ⁻²
Bi-205	Bismuth (83)	7.0×10 ⁻¹	1.9×10 ¹	7.0×10 ⁻¹	1.9×10 ¹	1.5×10 ³	4.2×10 ⁴
Bi-206	3.0×10 ⁻¹	8.1	3.0×10 ⁻¹	8.1	3.8×10 ³	1.0×10 ⁵
Bi-207	7.0×10 ⁻¹	1.9×10 ¹	7.0×10 ⁻¹	1.9×10 ¹	1.9	5.2×10 ¹
Bi-210	1.0	2.7×10 ¹	6.0×10 ⁻¹	1.6×10 ¹	4.6×10 ³	1.2×10 ⁵
Bi-210m (a)	6.0×10 ⁻¹	1.6×10 ¹	2.0×10 ⁻²	5.4×10 ⁻¹	2.1×10 ⁻⁵	5.7×10 ⁻⁴
Bi-212 (a)	7.0×10 ⁻¹	1.9×10 ¹	6.0×10 ⁻¹	1.6×10 ¹	5.4×10 ⁵	1.5×10 ⁷
Bk-247	Berkelium (97)	8.0	2.2×10 ²	8.0×10 ⁻⁴	2.2×10 ⁻²	3.8×10 ⁻²	1.0
Bk-249 (a)	4.0×10 ¹	1.1×10 ³	3.0×10 ⁻¹	8.1	6.1×10 ¹	1.6×10 ³