§ 52.135 Contents of applications; general information.
The application must contain all of the information required by 10 CFR 50.33(a) through (d) and (j).
A whole body dose of 25 rem has been stated to correspond numerically to the once-in-a-lifetime accidental or emergency dose for radiation workers which, according to NCRP recommendations at the time could be disregarded in the determination of their radiation exposure status (see NBS Handbook 69 dated June 5, 1959). However, its use is not intended to imply that this number constitutes an acceptable limit for an emergency dose to the public under accident conditions. Rather, this dose value has been set forth in this section as a reference value, which can be used in the evaluation of plant design features with respect to postulated reactor accidents, to assure that these designs provide assurance of low risk of public exposure to radiation, in the event of an accident.
during normal reactor operations described in 10 CFR 50.34a(e):

(1) The information pertaining to design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(2) An analysis and description of the equipment and systems for combustible gas control as required by §50.44 of this chapter;

(3) The list of electric equipment important to safety that is required by 10 CFR 50.49(d);

(4) A description of protection provided against pressurized thermal shock events, including projected values of the reference temperature for reactor vessel beltl ine materials as defined in 10 CFR 50.60 and 50.61;

(5) Information demonstrating how the applicant will comply with requirements for reduction of risk from anticipated transients without scram (ATWS) events in §50.62;

(6) The coping analysis, and any design features necessary to address station blackout, as described in §50.63 of this chapter;

(7) Information demonstrating how the applicant will comply with requirements for criticality accidents in §50.68(b)(2)–(b)(4);

(8) A description and analysis of the fire protection design features for the standard plant necessary to comply with part 50, appendix A, GDC 3, and §50.48 of this chapter;

(9) A description of the quality assurance program applied to the design of the SSCs of the facility. Appendix B to 10 CFR part 50, “Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants,” sets forth the requirements for quality assurance programs for nuclear power plants. The description of the quality assurance program for a nuclear power plant shall include a discussion of how the applicable requirements of appendix B to 10 CFR part 50 were satisfied;

(10) The information necessary to demonstrate how operating experience insights have been incorporated into the plant design;

(11) The information necessary to demonstrate how operating experience insights have been incorporated into the plant design;

(12) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(13) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(14) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(15) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(16) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(17) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(18) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(19) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(20) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(21) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(22) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(23) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(24) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(25) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(26) An analysis and description of design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof;

(b) An application for approval of a standard design, which differs significantly from the light-water reactor designs of plants that have been licensed and in commercial operation before April 18, 1989, or uses simplified, inherent, passive, or other innovative means to accomplish its safety functions, must meet the requirements of 10 CFR 50.43(e).

§52.139 Standards for review of applications.

Applications filed under this subpart will be reviewed for compliance with the standards set out in 10 CFR parts 20, 50 and its appendices, and 10 CFR parts 73 and 100.

§52.141 Referral to the Advisory Committee on Reactor Safeguards (ACRS).

The Commission shall refer a copy of the application to the ACRS. The ACRS shall report on those portions of the application which concern safety.